

Impact of gold ions irradiation on the initial alteration rate of the International Simple Glass

Célia Gillet, Stephanie Szenknect, Magaly Tribet, Hélène Arena, Sylvain Peuget

▶ To cite this version:

Célia Gillet, Stephanie Szenknect, Magaly Tribet, Hélène Arena, Sylvain Peuget. Impact of gold ions irradiation on the initial alteration rate of the International Simple Glass. Journal of Nuclear Materials, 2024, 588, pp.154817. 10.1016/j.jnucmat.2023.154817. cea-04844467

HAL Id: cea-04844467 https://cea.hal.science/cea-04844467v1

Submitted on 17 Dec 2024

HAL is a multi-disciplinary open access archive for the deposit and dissemination of scientific research documents, whether they are published or not. The documents may come from teaching and research institutions in France or abroad, or from public or private research centers.

L'archive ouverte pluridisciplinaire **HAL**, est destinée au dépôt et à la diffusion de documents scientifiques de niveau recherche, publiés ou non, émanant des établissements d'enseignement et de recherche français ou étrangers, des laboratoires publics ou privés.

Impact of gold ions irradiation on the initial alteration rate of the

International Simple Glass

C. Gillet^{a*}, S. Szenknect^b, M. Tribet^a, H. Arena^a and S. Peuget^{a*}

- ^a: CEA, DES, ISEC, DE2D, Univ Montpellier, Marcoule, France
- 7 b: ICSM, Univ Montpellier, CEA, CNRS, ENSCM, site de Marcoule, Bagnols sur Cèze, France

- *corresponding authors: celia.gillet@cea.fr (C. Gillet)
- 10 <u>sylvain.peuget@cea.fr</u> (S. Peuget)

Highlights

- ISG glass was irradiated with 7 MeV gold ions.
- Leaching experiments were performed on irradiated and non-irradiated samples.
 - Initial alteration rate of ISG glass increased by a factor of 2 after irradiation with Au ions.

Abstract

This study addresses the impact of 7 MeV Au irradiation on the initial alteration rate of International Simple Glass (ISG), considered as a reference for nuclear glasses. This irradiation scenario simulates the effect of nuclear collisions of the recoil nuclei of α decays. In order to study the very first moments of glass alteration, non-irradiated and Au ions irradiated ISG glass monoliths were altered in pure water at 90°C, pH 9 and with a ratio of glass surface area to solution volume of 0.01 cm⁻¹. Glass alteration was monitored by measuring the releases of the glass elements in solution by UV-visible spectrophotometry and Inductively Coupled Plasma-Atomic Emission Spectroscopy (ICP-AES). Under these alteration conditions, the releases of the elements in solution are congruent. The initial

- 27 alteration rate was found to be (2.0 ± 0.8) times higher for the irradiated glass monoliths than for non-
- 28 irradiated samples.

Introduction

France has chosen to reprocess spent fuel in order to save part of the uranium resources through the manufacture of MOx fuel. This recycling strategy also reduces the radiotoxicity of final waste as well as its volume. In order to find a management solution suitable to each type of nuclear waste, they are classified according to their level of radioactivity and lifetime. High-level waste (HLW) are packaged into a vitrified form and are intended for long-term disposal in a deep geological repository (Cigeo project). Once the waste packages are introduced into the disposal site, the glass will initially evolve into a closed system and will be subjected to its own self-irradiation due to the fission products and minor actinides incorporated in the glass structure. Fission products produce β decays accompanied by γ transitions that predominate with respect to the total activity of the waste in the short term (first 500 years), while α decays coming from actinide decays are the main contributor to the activity for the long-term [1]. α decays are responsible for the production of recoil nuclei (with an energy of about 100 keV) and α particles (with an energy of about 5 MeV). The glass is therefore submitted to dose rate and cumulative dose caused by these radiation sources, which can damage the glass by accumulation of radiation effects into the glass structure.

There are many studies [2-5] on the impact of irradiation on the structure and the macroscopic properties of borosilicate glasses including ISG glass (reference glass chosen by the international scientific community). These studies have shown that nuclear effects are predominant and that coupled effects between nuclear and electronic collisions can exist. Indeed, the α particles induce a partial annealing of the damage generated by the heavy ions. In addition, Angeli et al [6] studied the effect of hyperquenching on ISG glass fibers and reported a disordered structure similar to that of induced by irradiation.

After a few hundred or even a few thousand years, water will access the cells containing the waste packages. Then, the glass will evolve into an open system, which represents a risk of dissemination of radionuclides by water. It is thus of primary importance to assess the glass chemical durability. Once the glass package is in contact with the groundwater, exchanges of chemical species take place at the solid/solution interface leading to changes in the properties of the glass [7-9]. Several successive mechanisms occur at the glass/solution interface with different kinetics. Glass hydration by diffusion of water in its structure precedes the interdiffusion step, which consists of an ion exchange between the protons from the solution and the alkaline elements contained in a layer of a few angstroms at the glass surface. Then, hydrolysis of the vitreous network consists of a nucleophilic attack of a hydroxide ion OH⁻ on a bridging bond leading to a congruent dissolution of the glass [10]. The maximum alteration rate (called r₀) in a given environment is observed during the hydrolysis stage. The impact of the recoil nuclei of α decays on the glass initial alteration rate regime has already been partially addressed in the literature. Most studies are dedicated to glasses of complex composition and are rather descriptive and did not show any effect of radiation on the initial alteration rate [11, 12]. Recent studies, rather focused on simplified glasses, seem to indicate an impact of this type of irradiation at short alteration times [13-15]. However, these studies did not fill all the gaps. Indeed, the ISG glass irradiated with heavy ions studied by Karakurt, has been altered more deeply than the irradiated zone, the rate obtained is therefore not characteristic of the irradiated zone only. Zhang et al. studied a simple 3-oxide glass, as well as Lonartz et al. In addition, the ions used have a high energy whose representativeness of the effects of self-irradiation by α decay is not yet fully demonstrated and the alteration conditions do not quite correspond to an initial alteration regime. That's why this study focused on ISG glass (reference glass chosen by the scientific community), irradiated with 7 MeV gold ion irradiation in order to simulate the nuclear damage caused by recoil nuclei of α decays. In addition, the structural evolution of ISG submitted to this type of irradiation has already been described in the literature [3-5]. Leaching tests have been performed using low surface over solution volume ratio to determine the initial leaching rate of irradiated and non-irradiated ISG glass samples.

54

55

56

57

58

59

60

61

62

63

64

65

66

67

68

69

70

71

72

73

74

75

76

77

78

The current article presents the experimental conditions, the results of the leaching tests and discusses the origin of the effect of the external irradiation on the leaching rate.

Materials and methods

Glass samples preparation

In this study, the alteration experiments were carried out on the International Simple Glass (ISG) provided by MoSCI Corporation (Rolla, Missouri, USA). Its composition is given in Table 1 [8].

Table 1: ISG glass composition.

Oxide	SiO ₂	B ₂ O ₃	Na₂O	Al ₂ O ₃	CaO	ZrO ₂	
wt %	56.2	17.3	12.2	6.1	5.0	3.3	
mol %	60.1	16.0	12.7	3.8	5.7	1.7	
Element	Si	В	Na	Al	Са	Zr	0
wt %	26.3	5.4	9.0	3.2	3.6	2.4	50.1

Glass monoliths were obtained from a glass cylinder of 2 cm in diameter that was cut into monoliths which were polished to optical grade on both sides (up to reach a sample thickness of 700 μ m) and on the edge (Prime Verre company, Montpellier, France). Indeed, surface roughness may influence glass alteration therefore, it is necessary that the samples would be completely polished. The geometric surface area of the polished monoliths was considered in calculations of the normalized alteration rate.

Irradiation scenario

ISG glass monoliths were irradiated on both sides with 7 MeV Au³⁺ ions on the 2MV Aramis accelerator at the SCALP facility (CNRS-CSNSM, Orsay, France) to induce a significant nuclear damage in the glass structure as demonstrated in numerous studies that has been recently reviewed in [3]. A fluence of 2×10^{14} at.cm⁻² was used, which induces a minimum nuclear dose of around 40 MGy in the all the damaged depth [4] and thus to reach the saturation damage. The Au implantation profile was measured by negative ions analysis by Time-of-Flight Secondary Ion Mass Spectrometry (ToF-SIMS)

and was also obtained by simulation with the Stopping and Range of Ions in Matter (SRIM) software [16]. It was shown that the 7 MeV Au $^{3+}$ ions stop after a range of around 2 μ m within the glass [4], creating thus a damage depth of 2 μ m thick. It is worth noting that the alteration protocol was chosen to generate an altered depth less than or equal to the damaged depth so that the non-irradiated glass does not contribute to the initial alteration rate value. However, in addition to the two main irradiated faces, the edges of the sample which had not been irradiated and represented 10 % of the geometrical surface area contributed to the release of elements in solution. Thus, a correction was achieved when calculating the alteration rate (see equation 3).

Alteration protocol

The leaching experiments were carried out in PFA (Perfluoroalkoxy) reactors, Savillex® brand. These experiments were conducted on the non-irradiated ISG glass (named NI_glass later) and on the ISG glass irradiated with Au ions (named Au_glass later). The alteration experiments were performed in an oven at 90°C and at a pH_{90°C} of 9.0 ± 0.1. The solution was continuously stirred thanks to a magnetic stirrer. The volume of solution was adapted to the surface area of the coupon to reach the chosen surface over solution volume ratio (SA/V) of 0.01 cm⁻¹. The pH 9 solution was prepared by adding 9.53 mg of lithium hydroxide (99.995% purity, Alfa Aesar) into 1 L of deionized water. The pH was measured once the solution was prepared, at 90°C, under strong stirring and reached 9.0 ± 0.1. Then, the lithium hydroxide solution was introduced into the Savillex® reactor that was placed for the night in an oven at 90°C the day before the start of the experiment. At the beginning of the experiment, a PFA sample holder was held in the middle of the leachate, using a Teflon ribbon. The glass monolith was finally placed on this sample holder so it would be completely immersed in the solution. The complete setup is shown in Figure 1.

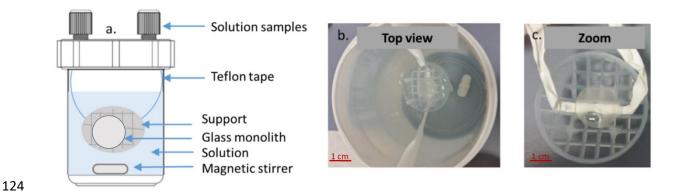


Figure 1 : Schematic representation of a Savillex $^{\circ}$ reactor (a), top view of the reactor (b) and zoom on the glass monolith on its holder (c).

The alteration experiment lasted 24 hours but the initial alteration rate was calculated over the first 8 hours to guarantee a concentration in silicon less than 1 mg.L $^{-1}$. This choice was made to avoid the influence of saturation phenomenon of the solution with respect to silica on the alteration rate [17]. In addition, for Au_samples, beyond 8 hours of alteration, the altered thickness became greater than the irradiated depth (2 μ m). Moreover, to limit disruptions of the system, the number and the volume of solution were adapted to limit the total sampled volume (less than 20 % of the total volume). Experiments on NI_glass and Au_glass were repeated 3 times each.

Solution analyses

Under these conditions, all the elements coming from the glass are expected to be released in solution (except zirconium which is very poorly soluble at a pH of 9.0) [18] and can be used to determine the alteration rate. Measurements of the elemental concentrations were conducted using both UV-visible spectrophotometry (Cary Varian 50 Scan) and ICP-AES (Spectro Arcos EOP). For UV-visible spectroscopy measurements, commercial Spectroquant® kits were used to prepare the standard solutions and unknown samples. The absorbance was measured at a wavelength of 665 nm for silicon and 405 nm for boron using a UV-visible spectrophotometer. The absorbance measurement was repeated 5 times to check the stability of the signal and an average of the 5 measurements was

retained. Calibration was done using several standard solutions prepared by dilution in deionized water of commercial standard solutions at 1 g.L⁻¹ (ARISTAR® single element calibration standard for ICP-AES 1000 mg L⁻¹, VWR). The concentrations of the standard solutions were in the range from 0.1 to 2 mg.L⁻¹ for B and from 0.5 to 5 mg.L⁻¹ for Si. 9 mL of each sample was used for UV-visible measurements and the remaining leachate sample (1 mL) was then acidified by adding a small amount (about 70 μ L) of 14 N nitric acid (Suprapur®,14 N, 65%) to reach [HNO₃] = 1 mol.L⁻¹ and analyzed by ICP-AES. All analyses were carried out the day after the end of the experiment. 2 to 4 wavelengths per element were selected to measure the elemental concentration with ICP-AES. For each wavelength, calibration curves were determined from the analysis of several standard solutions prepared by dilution in 1 mol.L⁻¹ HNO₃ solution of certified standard solutions at 1 g.L⁻¹ (PlasmaCAL single element calibration standard for ICP-AES 1000 mg L⁻¹, SCP-Science). 3 measurements were done for each selected wavelength. Then, the elemental concentrations were calculated as the average of the results obtained for all wavelengths.

The relative experimental uncertainty on the concentration obtained by using UV-visible spectrophotometry was estimated to 10 %. For ICP-AES analyses, it corresponded to twice the relative standard deviation of the concentrations obtained for the different selected wavelengths (i.e. 5 % for B, Na and 10 % for Ca and Al).

The normalized mass losses (*NL*_i(t)) for element *i* at time *t* were calculated from the concentrations measured by UV-visible spectrophotometry or by ICP-AES, according to equation 1.

163
$$NL_i(t) = \frac{C_i(t) \times V(t) + \sum_{1}^{t-1} [C_i(t) \times V_j]}{x_i \times S} \text{ with } V(t) = V_0 - \sum_{1}^{t-1} V_j$$
 (1)

where: $C_i(t)$ is the concentration of element i measured in solution at time t (in g.L⁻¹); V(t) is the volume of solution remaining in the Savillex® reactor at time t (in L); V_j is the solution volume removed from the reactor at each sampling time (in L); V_0 is the initial volume of solution in the reactor (in L); x_i is the mass fraction of element i in the glass and S is the geometric surface area of the glass (in m²).

The alteration rate (*r*_i) for a given element *i* corresponds to the time derivative of the normalized mass loss (equation 2). It is usually expressed in g.m⁻².d⁻¹.

$$170 r_{\rm i} = \frac{d \left(NL_{\rm i}(t)\right)}{dt} (2)$$

The releases of the different elements of the glass were found to be congruent. Therefore, the effective rate, $r_{\text{effective}}$ was calculated by averaging the results obtained for the various replicated leaching experiments and all the elements (Si, B, Na, Ca, Al).

Moreover, a correction was performed to calculate the initial alteration rate in order to take into account that 10 % of the geometric surface area (i.e. the edges) of the monolith was not irradiated.

The corresponding initial alteration rate of a monolith entirely irradiated with Au ions, r(Au_glass), was calculated using equation (3):

178
$$r(Au_glass) = \frac{r_{effective} - 0.1 \times r(NI-glass)}{0.9}$$
 (3)

where $r(NI_glass)$ is the alteration rate of the non-irradiated glass and $r_{\rm effective}$ is the alteration rate calculated from the experimental normalized mass losses of the monolith irradiated on the two main faces.

Results

alteration. This regime is controlled by hydrolysis of the vitreous network [10] and corresponds to a maximum rate of alteration of a glass in a given medium.

From the releases in solution, the normalized mass losses were calculated for all the elements of the glass and plotted as a function of the alteration time for the 6 samples in Figure 2. It is worth noting that over a period of 8 hours, the evolution of normalized mass losses was linear, regardless of the

The study of the initial alteration rate regime allows to focus on the first moments of the glass

element considered and the type of sample considered (NI_glass or Au_glass). Congruency was systematically observed for B, Al and Ca, while normalized mass losses in Na were systematically higher. For Ca, outliers were attributed to potential contaminations (points drawn in red and framed) and these data were not considered in the determination of the r_{Ca} . For NI_1 and Au_1 tests, the releases of Ca were too scattered, therefore they were not considered in the subsequent calculations of r_{Ca} . For Si and Na, the initial concentrations are variable. It was thus assumed that an initial contamination in the Savillex® reactor could have induced an initial offset of the concentrations. This initial offset had virtually no impact on the slope of the $NL_{Si}(t)$ and $NL_{Na}(t)$ curves. Thus, these data were used to determine r_{Si} and r_{Na} .



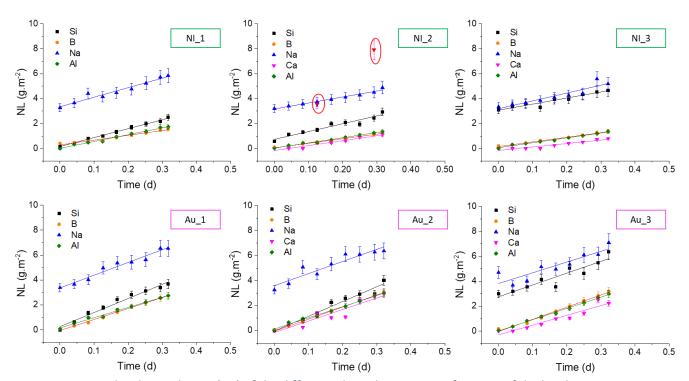


Figure 2: Normalized mass losses (NL) of the different glass elements as a function of the leaching time for the 3 experiments with NI_samples (NI_1,2,3) and with Au_samples (Au_1,2,3). The data circled in red were not used in the calculation of r_{Ca} .

Linear regression was performed on the NL obtained for each element and the slopes corresponding to the initial rates are reported in Table 2. We observed that, regardless of the element considered and the series, the results of the various experiments were not significantly different. The initial alteration rates calculated were systematically lower for NI_glasses than for irradiated Au_glasses. For NI_glasses, an average initial rate of $(4.9 \pm 3.0) \text{ g.m}^{-2}.d^{-1}$ was obtained compared to $(10.0 \pm 2.5) \text{ g.m}^{-2}.d^{-1}$ for the glasses irradiated with Au ions. Therefore, there is a significant impact of Au ions irradiation on the value of the initial alteration rate. This impact led to an increase in the average initial alteration rate by a factor $F = 2.0 \pm 0.8$. This higher initial alteration rate in the case of ISG glass samples irradiated with Au ions indicated a difference in chemical reactivity at the very first moments of alteration.

Table 2: Initial alteration rate in g.m⁻².d⁻¹ for each element of the glass and for each test (r_i) . The ratio of the average initial alteration rates of irradiated samples to non-irradiated samples $(F = r(Au_glass)/r(Nl_glass))$ is given in the last column of the table. The final rate (given in g.m⁻².d⁻¹ and μ m.d⁻¹) is the average of all the elements and all the tests. The $r_{effective}$ value is the calculated leaching rate without considering the non-irradiated edges. All uncertainties correspond to twice the standard deviation except for r(Au) where $\Delta r(Au) = \Delta r_{effective} + 0,1 \times \Delta r_{Nl}$.

<i>r</i> _i (g.m ⁻² .j ⁻¹)	NI_1	NI_2	NI_3	Au_1	Au_2	Au_3	F
Si	7.1	6.3	4.9	11.3	11.8	9.5	1.8
В	3.8	3.6	3.7	8.8	9.7	9.8	2.5
Na	7.6	4.8	6.3	10.2	9.7	8.6	1.5
Ca		3.9	2.8		9.2	7.8	2.2
Al	5.5	4.1	3.9	8.2	8.9	9.0	1.9
r _{effective} (g.m- ² .d ⁻¹)			9.5 ± 2.2				
r (g.m- ² .d ⁻¹)	4.9 ± 3.0 10.0 ± 2.5						
r (μm.d ⁻¹)	2.0 ± 1.2 4.0 ± 1.0						

Discussion

233234

235

Initial alteration rate values of ISG glass

236

237

238

239

240

241

242

243

244

245

246

247

248

249

250

251

252

253

254

255

256

This study allows to determine the initial alteration rate of ISG at 90°C. It is important to note that only few data regarding the value of the initial alteration rate of ISG glass exist in the literature and that the published data are variable (see Table 2). Most of these data concern non-irradiated ISG specimens. Some studies were conducted on ISG glass powder (the surface area was measured by the BET method or using the geometric surface of the powder) and others on monoliths. In order to be able to compare the initial alteration rate values obtained for powders and for monoliths, it is necessary to use corrective factors as assessed by Fournier et al. [17]. Indeed, the initial alteration rate obtained from the powder should be multiplied by a factor of 0.8 when the surface area considered is the geometric one and by 1.9 when the surface area is measured by the BET method. Jegou [19] studied the initial alteration rate of non-irradiated ISG glass powder altered in pure water at 90°C, with SA/V = 0.1 cm⁻¹ and pH = 9, over 2 days. Jegou reported an initial alteration rate of r_0 = 1.7 g.m⁻².d⁻¹, which becomes r_0 (monolith) = 3.2 g.m⁻².d⁻¹, using the corrective factor recommended by Fournier et al. Gin et al. [20] also altered ISG glass powder in initial rate regime in pure water at pH 9, 90°C, with a SA/V ratio of 0.076 cm⁻¹ and for 5.7 h. They obtained a r_0 value of 8.2 \pm 2.5 g.m⁻².d⁻¹. After applying the appropriate corrective factor of 0.8, the corresponding average rate for a monolith is 6.6 g.m⁻².d⁻¹. Inagaki et al. [21] carried out leaching experiments at different pH and temperatures on ISG glass monoliths and determined a relation between the alteration rate at 90°C and the pH (log $r_{\rm Si}$ = -1.56 +

0.236 × pH). The use of this relation for pH = 9 gave r_{Si} = 3.7 g.m⁻².d⁻¹.

Thus, considering data from the literature, the initial alteration rate of a non-irradiated ISG glass in pure water at pH 9 and 90°C ranges from 3.2 to 6.6 g.m $^{-2}$.d $^{-1}$. The value obtained in this work for non-irradiated ISG monoliths (i.e. 4.9 ± 3.0 g.m $^{-2}$.d $^{-1}$) is well included in this range. This observation led to the conclusion that the experimental protocol and the data treatment were reliable.

Table 2: r_0 values of ISG glass reported in the literature and from the current study.

2	64

References	рН	Temperature (°C)	S/V (cm ⁻¹)	Powder or monolith?	Corrective factor	Leaching protocol	Time (d)	Corrected r ₀ (g.m ⁻² .d ⁻¹)
Jegou 1998	9	90	0.1	Powder	×1.9	Static	2	3.2
Gin 2020	9	90	0.076	Powder	×0.8	Static	0.24	6.6 ± 2.5
Inagaki 2013	9	90		Monolith		Flow cell		3.7
This study	9	90	0.01	Monolith		Static	0.33	4.9 ± 3.0

To the best of our knowledge, there are only two studies reporting results on the initial alteration rate of ISG glass whose structure was damaged before alteration. Angeli et al. [6] studied ISG glass fibers hyperquenched (simulating structural disorder close to the one induced by irradiation) and therefore structurally more disordered. They showed that the silicate network, was hydrolyzed 1.4 to 1.8 times faster than for annealed fibers. This result is close to the increase factor obtained in this study, which is 2.0 ± 0.8 . However, the second study of Karakurt et al. [13] on the initial rate of ISG glass irradiated with gold ions cannot be considered since the glass was altered deeper than the irradiated zone. Thus, the results obtained in our study brings new information and showed an effect of the glass

Thus, the results obtained in our study brings new information and showed an effect of the glass disorder on its initial alteration rate and is consistent with the literature: the r_0 of an irradiated or hyperquenched ISG glass is increased compared to the one of a non-irradiated ISG glass.

278

279

280

281

282

283

284

285

286

287

288

289

290

291

292

293

294

295

296

297

298

299

300

301

Origin of the rate difference between NI_glass and Au_glass

The initial alteration rate of Au_glass samples was found to be higher than that of NI_glass by a factor of 2.0 ± 0.8. This observation means that the chemical reactivity of the Au_glass with respect to the solution (pure water) is greater than that of the NI_glass during the hydrolysis step. Several authors [5, 6, 22, 23] showed that there is a link between the structural disorder of glass, whether thermally induced [6] or by irradiation [5, 22, 23], and its chemical durability in the long term alteration rate regime. The current study demonstrated that this is also true for the initial alteration rate of ISG glass. Several explanations could be proposed to explain the increase of r₀ due to the glass disorder induced by Au ions irradiation. Firstly, radiation damage as well as high quenching rate induce a modification of the glass short range order with the creation of punctual defects and non-bridging oxygen atoms that induce a decrease of the polymerization of the glass network. Moreover a partial conversion of BO₄ in BO₃ species [2, 4-6, 14, 24] is observed that also participates to the decrease of the glass polymerization degree which could induce a higher hydrolysis of the silicate network and result in an increase in glass alteration rate. Overall, the general depolymerisation of the glass network results in a lower number of bridging bonds between network former atoms. The initial rate regime is controlled by the hydrolysis of the vitreous network through the breaking of its bridging bonds (Si-O-Si, Si-O-Al or Si-O-Zr for example) by protonation or hydroxylation of reactive surface sites [10]. Thus, the hydrolysis of the glass can be favoured by a low degree of polymerization of the vitreous network. This was recently confirmed by Monte Carlo simulations which showed that the primary cause for the increase of glass alteration rate was the extent of the depolymerization of the glass network introduced by these local structural changes (boron coordination, number and nature of Qⁿ species, and angular distribution of Si-O-Si linkages) [25]. Moreover the reactivity of B(III) was shown to be higher than that of B(IV). Kapoor et al. [26] showed that the boron (+III) could be seen as a Lewis acid, with water acting as a Lewis base.

Moreover, by considering the Lewis acido-basicity of borate species, an increased reactivity of the B(+III) compared to B(+IV) can be assumed to increase the glass network hydrolysis.

Secondly, the radiation damage generated by nuclear collisions also induces a modification of the glass medium range order (MRO) with mainly a broadening of the ring size and angle distribution [5, 27], as indicated by the increase of three membered silicate rings and the shift of R band. The MRO modification was also observed by molecular dynamic simulation of radiation damage and high quenching rate in simple borosilicate glasses [28]. This modification of the ring size distribution directly affects the glass free volume and thereby the accessibility of the water into the glass network [28], that may also favour its hydrolysis.

Finally, these mechanisms can occur either upstream of the hydrolysis step, the water penetrates faster into the glass and therefore leads to a deeper hydrolysis [29] or are directly related to the increase of B(III), more reactive towards water [24, 26]. These two mechanisms can effectively act concomitantly and synergistically.

Comparison between ISG and SON68 complex nuclear glass

In the literature, it was shown for SON68 glass that irradiation with Au ions had no significant impact on the initial alteration rate [11]. Indeed, the initial rate of SON68 glass was about 2 g.m⁻².d⁻¹, whether the glass was irradiated with Au ions or not [11]. This result was also observed for SON68 glasses doped with actinides [11, 12, 30]. The more complex chemical composition of SON68 glass compared to ISG glass could certainly explain the weaker impact of Au ions on the glass structure and thus on the initial alteration rate [3]. More experiments on complex SON68 glass are needed to quantify its structural evolution versus irradiation and compare it to the one of ISG glass so as to try to explain the lower sensitivity of SON68 glass initial alteration rate under irradiation.

Conclusion

This study was dedicated to the impact of nuclear collisions (simulating the effects of the recoil nuclei of α decays) on the initial alteration rate of the ISG glass. It was shown that the initial alteration rate of the ISG glass irradiated with 7 MeV Au ions was 2.0 ± 0.8 times higher than that of a non-irradiated ISG glass. This work demonstrated that there is a link between the structural changes of ISG glass caused by nuclear collisions and its initial alteration rate. The origin of the increase of the alteration rate seems to be associated to the short and medium range order changes induced by the nuclear collisions, and certainly by the amplitude of the boron coordination change. Similar irradiation conditions on SON68 glass did not induce any changes of its alteration rate which suggests that the more the structure of the glass is modified by irradiation (change in boron coordination, change in NBO), the higher the alteration rate of the glass is.

Subsequently, it would be interesting to continue the investigation of the structural changes induced by irradiation on more complex glasses, up to SON68 one, in order to identify the glass composition domain that did not show any dependence of its initial alteration rate with irradiation.

Acknowledgments

The authors would like to thank Pascale Colombel and Laurent Duffours (Prime Verre company, Montpellier, France) for sample preparation, Jérôme Bourçois (CNRS-CSNSM, Orsay, France) and Olivier Cavani (Ecole Polytechnique, Palaiseau, France) for carrying out irradiations, Elodie Chauvet and Loan Lai (Tescan Analytics, Fuveau, France) and Laurent Dupuy (SERMA Technologies, Ecully, France) for the ToF-SIMS characterizations. This work was performed in the framework of the Vestale project co-funded by EDF, CEA and Orano and it was supported by INWARD coordinated research project [Ion Beam Irradiation for High Level Nuclear Waste Form Development, F11022] from IAEA.

References

- Weber, W.J., et al., *Radiation effects in glasses used for immobilization of high-level waste* and plutonium disposition. Journal of Materials Research, 1997. **12**(8): p. 1946-1978.
- Peuget, S., J.M. Delaye, and C. Jegou, *Specific outcomes of the research on the radiation* stability of the French nuclear glass towards alpha decay accumulation. Journal of Nuclear Materials, 2014. **444**(1-3): p. 76-91.
- Mir, A.H. and S. Peuget, Using external ion irradiations for simulating self-irradiation damage
 in nuclear waste glasses: State of the art, recommendations and, prospects. Journal of
 Nuclear Materials, 2020. 539: p. 152246.
- Gillet, C., et al., *Impact of complex irradiation scenarios on the structure and the properties of the International Simple Glass.* Journal of Nuclear Materials, 2022. **572**: p. 154079.
- Peuget, S., et al., Radiation effects in ISG glass: from structural changes to long term aqueous
 behavior. npj Materials Degradation, 2018. 2:23.
- Angeli, F., et al., *Effect of thermally induced structural disorder on the chemical durability of International Simple Glass.* npj Materials Degradation, 2018. **2**(1): p. 31.
- Frugier, P., et al., SON68 nuclear glass dissolution kinetics: Current state of knowledge and basis of the new GRAAL model. Journal of Nuclear Materials, 2008. **380**(1-3): p. 8-21.
- 372 8. Gin, S., et al., *An international initiative on long-term behavior of high-level nuclear waste* 373 *glass.* Materials Today, 2013. **16**(6): p. 243-248.
- Gin, S., et al., *Insights into the mechanisms controlling the residual corrosion rate of borosilicate glasses.* npj Materials Degradation, 2020. **4**(1): p. 41.
- 376 10. Gin, S., et al., *The controversial role of inter-diffusion in glass alteration*. Chemical Geology, 377 2016. **440**: p. 115-123.
- Peuget, S., et al., *Effect of alpha radiation on the leaching behaviour of nuclear glass.* Journal of Nuclear Materials, 2007. **362**(2-3): p. 474-479.
- Tribet, M., et al., *Irradiation Impact on the Leaching Behavior of HLW Glasses.* Procedia Materials Science, 2014. **7**: p. 209-215.
- 382 13. Karakurt, G., *Effect of alpha radiation on the physical and chemical properties of silicate* 383 *glasses*. 2014, Ecole des Mines de Nantes.
- Lonartz, M.I., et al., The Effect of Heavy Ion Irradiation on the Forward Dissolution Rate of
 Borosilicate Glasses Studied in Situ and Real Time by Fluid-Cell Raman Spectroscopy.
 Materials, 2019. 12(9): p. 1480.
- 387 15. Zhang, X.Y., et al., *Influence of ion radiation on leaching behavior of borosilicate glass.* Journal of Non-Crystalline Solids, 2023. **602**: p. 122091.
- Ziegler, J.F., J.P. Biersack, and M.D. Ziegler, SRIM, the stopping and range of ions in matter.
 2008, Chester, Maryland: SRIM Co.
- Fournier, M., et al., Glass dissolution rate measurement and calculation revisited. Journal of
 Nuclear Materials, 2016. 476: p. 140-154.
- 393 18. Brown, P.L., E. Curti, and B. Grambow, *Chemical Thermodynamics of Zirconium*, ed. C.T. 8. 2005.
- Jegou, C., Mise en évidence expérimentale des mécanismes limitant l'altération du verre R7T7
 en milieu aqueux. Critique et proposition d'évolution du formalisme cinétique. 1998, Thèse de
 l'Université Montpellier II. p. [330 p.] Pagination multiple.
- 398 20. Gin, S., et al., *Can a simple topological-constraints-based model predict the initial dissolution* 399 rate of borosilicate and aluminosilicate glasses? npj Materials Degradation, 2020. **4**(1): p. 6.
- 400 21. Inagaki, Y., et al., *Initial Dissolution Rate of the International Simple Glass as a Function of pH*401 and Temperature Measured Using Microchannel Flow-Through Test Method. International
 402 Journal of Applied Glass Science, 2013. **4**(4): p. 317-327.
- 403 22. Mougnaud, S., et al., *Heavy ion radiation ageing impact on long-term glass alteration* 404 *behavior.* Journal of Nuclear Materials, 2018. **510**: p. 168-177.

- Tribet, M., et al., Alpha dose rate and decay dose impacts on the long-term alteration of HLW nuclear glasses. npj Mater. Degrad., 2021.
- 407 24. Stone-Weiss, N., et al., *Understanding the structural drivers governing glass-water*408 *interactions in borosilicate based model bioactive glasses.* Acta Biomaterialia, 2018. **65**: p.
 409 436-449.
- 410 25. Jan, A., et al., *Monte Carlo simulation of the corrosion of irradiated simplified nuclear waste* 411 *glasses.* Journal of Non-Crystalline Solids, 2019. **519**: p. 119449.
- 412 26. Kapoor, S., et al., Structural and Chemical Approach toward Understanding the Aqueous
 413 Corrosion of Sodium Aluminoborate Glasses. Journal of Physical Chemistry B, 2018. **122**(48):
 414 p. 10913-10927.
- 415 27. Karakurt, G., et al., *Understanding of the mechanical and structural changes induced by alpha*416 particles and heavy ions in the French simulated nuclear waste glass. Journal of Nuclear
 417 Materials, 2016. **475**.
- Delaye, J.M., et al., Comparative effects of thermal quenching and ballistic collisions in SiO2 B2O3-Na2O glass. Nuclear Instruments & Methods in Physics Research Section B-Beam
 Interactions with Materials and Atoms, 2014. 326: p. 256-259.
- 421 29. Mansas, C., et al., *Drivers of Water Transport in Glass: Chemical or Topological Effect of the Glass Network?* The Journal of Physical Chemistry C, 2017. **121**(30): p. 16201-16215.
- Wellman, D.M., J.P. Icenhower, and W.J. Weber, *Elemental dissolution study of Pu-bearing borosilicate glasses*. Journal of Nuclear Materials, 2005. **340**(2-3): p. 149-162.

425