

Overview of the EUROfusion Tokamak Exploitation Programme in support of ITER and DEMO

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EURO*fusion*

Overview of the EUROfusion Tokamak Exploitation Programme in support of ITER and DEMO

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and the EUROfusion Tokamak Exploitation Team, the ASDEX Upgrade team, the JET contributors, the MAST-U team, the TCV team and the WEST team

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4- Machine generic integrated control

Disruption proximity control in AUC (left) and LFT (right): The upper triangularity δ_{up} is an effective actuator for the H-mode density limit (HDL) disruption avoidance on AUG [10]. Automation of the dynamic pulse schedule has proven successful to scan the

X-point Radiation control in AUG [13]: The XPR is induced by impurity seeding during the

heating phase. It can be monitored in real-time and regulated by nitrogen injection. The active XPR nearing prase, is can be indimined on treat-line and retaining detachment in mouton. The active APPR control enabled the first demonstration of maintaining detachment throughout a full discharge including the transitions in and out of H-mode, featuring full detachment with a 80% radiation fraction. The ELM suppression, connected to a high location of the XPR, is also observed in the whole parameter range. The XPR can now routinely be reproduced in SOLPS simulations for AUG [14] and TCV [15]. Detachment control is now totalised in TCV, WEST (in L-mode) and JET.

6- Preparation of efficient PFC operation

Studies of poloidal gaps: Right: infrared image of the plasma facing unit with 1.5mm misalignment in WEST reproduced from [17]. Left: zoomed-in final deformation profile near the gap in AUG reproduced from [18]. MEMENTO modelling is iniciating that the melt will relatively freely cross the gaps without penetrating into the gap [19]. Experiment in WEST, using a dedicated PFU with expected PFU leading edge has allowed the benchmarking the

ulse schedule has prover of the HDL without disru

5- Physics of detachment

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EUROfusion Tokamak Exploitation programme developed within the European Fusion Roadmap: ITER DEMO

RT01: Core-Edge-SOL

integrated H-mode

- Prepare ITER exploitation
- Guide DEMO design
- Exploit Plasma EXhaust (PEX) machine enhancements



Physics & Control integration

5 devices:

ASDEX Upgrade (AUG), MAST-U, TCV, WEST and JET (2022-2023) Tokamak exploitation operates a fraction of typically 30 to 45% of the time of each machine, except JET: 100%

1- Core-Edge-SOL integrated H-mode



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2- Alternative to type-I ELM regimes



AUG separatrix operational space in edge density $n_{extraps}$ and temperature $T_{excraps}$ of the data set with Bit = 2.5 T and p = 0.8 MA. Open bulk symbols represent Quasi Continuous Exhaust QCCE) phases, filled red symbols represent type-I ELMy phases. q_{i} turbulent praneter $\approx q_{i} e^{-q_{i}}_{i}$ in black; ideal Ballooning Limit (IBL). Reproduced from (4). QCE scenario is being developed at JET using the same principles.

3- Disruption and run-away mitigation strategies

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-56 3



 H_{α} cloud recorded by the JET fast camera at the time of the pellet penetration and Thomson scattering edge measurement. The overlaid equilibrium is the last prior to the pellet arrival Reproduced from [7].

tiges, Spain, 21st June 2021 roud et al., 28th IAEA, Fusion Energy Conference, 1 and 48th EPS Conference on Plasma Physics aastricht, Netherlands, 27th June 2022. itschet al.2023 Nucl. Fusion 63 076013 writer et al., 49th European Conference on Plasma (EPS), [4]: M. Faitsch [5]: O. Février Physics (EPS) net al 2023 Nucl. Fusion 63 076013 r et al, 49th European Conference of 2023), Bordeaux, France, July 6th, 2 h et al, 2018 Plasma Phys. Control.

 B. J. Lohens et al., "Theory and Stratistics of Describes Phoneshop(TEOP), Phys. (Phys. Rev. 16, 144, 2023. Phys. (EP) 2020; Phys. (Phys. Rev. 164, 2023. Theorem 2014, "Athentic Phys. Rev. Phys. Rev. Phys. Phys. (EP) 2020; Phys. Rev. Phys. Rev. Phys. Phys. Rev. Phys. Rev. Phys. Rev. Phys. Rev. Phys. Phys. Rev. Phys. Phys. Rev. Phys. Rev. Phys. Phys. Rev. Phys. Rev. Phys. . on 60

SPI experiments also started in

 10^{2}



RT02: Alternative to

type-I ELM regimes

3MA/2.8T DT pulse in JET (DTE3 campaign) demonstrating core edge integration: with partial detachment via the control of target tile temperature with Neon seeded, small ELMs (ΔT=20C), at high plasma current (3MA) and low collisionality.



ð

2.5 MA/4 M

Ne atoms The attorns Single Ne/D SPI: No impact of velocity on current quench (CQ) duration → similar assimilation [8] Staggered SPI: CQ unchanged despite Ne is injected into dilution cooled target (T_a a few 100 eV) → lower Ne assimilation may be

compensated by higher density Dual Ne/D SPI: Faster CQ → higher assim

1022



Time is) SPI can prevent the toroidal asymmetrical phase of the Vertical Displacement Event (VDE), i.e. AVDE and the exponential growth of the plasma current centroid vertical position (2c) is interrupted and then stabilized Reproduced from [9]. The vessel reaction forces are strongly reduced in mitigated VDEs and that the difference in the energy deposited to the berylium upper dump plates is about 5 times less than for an unnitigated VDE.

21 Nucl. Fusion 61 024001 th International Conference on Plasm Components for Fusion Applicatio any, 22nd May 2023 clear Fusion, Volume 63, Number 6

(15) G. San et al. 40% European Conference Physics (26% 2023). Bordsaux, Francis, ed July 20 (16): M. Bernert et al 2021 Nucl. Fusion 61 024001 (17): V. Core et al. 10% International Conference (PFRIC-10), Born, Germany, 2011 May 2023 (19): K. Krisger et al. Nuclear Fasion, Volume 63, N. (19): S. Kahynchais et al to appear in Nucl. Fus. (19): S. Kahynchais et al. 20% International Conference Surface Interaction in Controlled Fusion Devices (P South Krens, 120): June 2022.

Sarnas Surface Interaction in Controlled Fusion D South Kreen, 21th June 2022. [23]: S. Coda et al, 29th IAEA Fusion and Control United Kingdom. Bith Colber 202 atlons: 24]: Y. Kazakiro et al, 29th IAEA Fusion London, United Kingdom. Bith Colber 202 [25]: J. Garcia, et al, 29th IAEA Fusion London, United Kingdom. Bith Colber 202 [26]: J. Otim and P. Mantica 2023 Plasma Lawing Market and P. Mantica 2023 Plasma -1939]

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Breakdown studies (JET): Toroidal (loop) electric field versus prefill gas pressure, both at the time of breakdown for lon Cyclotron assisted breakdown discharges with $B_T = 1.7T$ (red squares), $B_T = 2.0T$ (bleak squares), and $B_T = 2.3T$ (black squares), Reproduced from [12] Equips 0.33 V/m with BT = 1.7T is achieved in conditions which EC breakdown assistance with 170 GHz waves would not work in TER. Experiments also on-going in WEST and AUG.

6; = 2.5 T
 6; = 2.0 T
 6; = 1.7 T
 x failed play
 * Holasma

JET

p (mPa)

Igro

n=1 B, (mT)





XPR control in JET with Ne & Ar: The XPR moves inside the confined region, this movement can be controlled [16] bolometry. ELMs are also suppressed. Detachment control achieved successfully in DTE3. Vertical position is tracked by the div bolometry. ELMs are also suppres



noval after JET DTE2 Camp ent methods deduced diff subdivertor residual gas analysis data baking: 2- 240C baking (extrapolated baking; 2- 240C baking (extrapolated); ICWC & plasma inner divertor deposits; Baking dusing ICWC; 5- glow discharge; 6-total. Reproduced from [20]. Baking removes

scaling with R



MEMENTO code under a regime of shallow melt

Alternative divertor studies with long legs shapes

(B. Lipschultz et al Nucl. Fusion 56 (2016) 056007 In TCV: CIII front movement (and LPs) show partially reduced detachment threshold with increasing Leg length. Reproduced from [21]
 In MAST-U: Near-ideal detachment threshold reduction in extreme, more strongly baffled MAST-U divertor. Reproduced from [22]

8-Physics of high β long pulse scenario



Development of the high $\beta_{\rm H}$ scenario in TCV and Mast-U: experimental database achieved with NBI on both devices (+ECRH in TCV) showing the confinement enhancement factor with color denoting the bootstrap c fraction. This work is due to continue in current and future campaigns [23].

Conclusion: The EUROfusion Tokamak Exploitation Work Package coordinated experiments between the 5 devices (AUG, JET, MAST-U, TCV and WEST) having different but complementary capabilities have demonstrated the benefit of this approach for answering key questions for ITER such as disruption mitigation, detachment control or the qualification of ITER grade components.

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 Negative triangularity (NT) studies in TCV: Temperature peak at the outer strike point higher in NT than in positive triangularity (PT) [5]. Experiments and simulations points towards lower λ_q in NT configuration L-Mode compared to PT L-Mode [6].

 Power exhaust in NT remains challenging: \diamond Topically, shorter connection length \diamond Indication of a smaller heat-fluw width \diamond Icower divertor pressure, hinting at different particle confinement





> [m⁻³]

TCV

Experimental results from TCV show weaker



validated yet

9- Physics of energetic particles