

Assessment of a bolted-flange connection for ITER Test Blanket Module

Tony Zaouter, Karl Vulliez, Benjamin Michel, S. Gazotti, J.-P. Friconneau,

Jean-Pierre Martins

► To cite this version:

Tony Zaouter, Karl Vulliez, Benjamin Michel, S. Gazotti, J.-P. Friconneau, et al.. Assessment of a bolted-flange connection for ITER Test Blanket Module. Fusion Engineering and Design, 2021, 165, pp.112247. 10.1016/j.fusengdes.2021.112247. cea-03720009

HAL Id: cea-03720009 https://cea.hal.science/cea-03720009

Submitted on 11 Jul2022

HAL is a multi-disciplinary open access archive for the deposit and dissemination of scientific research documents, whether they are published or not. The documents may come from teaching and research institutions in France or abroad, or from public or private research centers. L'archive ouverte pluridisciplinaire **HAL**, est destinée au dépôt et à la diffusion de documents scientifiques de niveau recherche, publiés ou non, émanant des établissements d'enseignement et de recherche français ou étrangers, des laboratoires publics ou privés.



Problem position

The Test Blanket Module (TBM) are core components of ITER. Several designs of tritium Breeding Blankets are proposed. All these concepts share a common feature: they have to be connected to a network of pipes, referred to as `Pipe Forest (PF)'. The PF connects the AEU to the TBM and will provide fluids for tritium process and coolant (gas/water).



The PF design is strongly constrained. In addition to stringent definition requirements, the PF has to be conceived under ALARA principles. In maintenance phase, the hands-on operational time required for connection/disconnection of the PF shall be minimized and Remote Handling (RH) operation will be considered when possible.

Two technical solutions are considered at the TBM/PF interface: welded and bolted flange connection. The present study focuses on the latter.

Mechanical design & Numerical model

Among the different concepts, we focus on three fluid lines: lithium-lead, water and helium.

regarding ITER safety Leak-rate criterion defined requirements and conditions:

- Liquids: `bubble tight' crit. with HELICOFLEX[®] seal
- Gas: `helium tight' crit. with double torus HELICOFLEX[®] seal for leak monitoring

Tab. 1: Design conditions

| | Lithium-Lead | Water | Helium |
|---------------------|----------------|----------------|----------------|
| Pipe size | DN25 | DN65 | DN80 |
| Pipe material | Eurofer | SS-316L | SS-316L |
| Outer diam. [mm] | 33.7 | 76.1 | 88.9 |
| Thickness [mm] | 5 | 12.5 | 8.8 |
| Flange `Class' | 300 | 2,500 | 2,500 |
| Bolts | 4 × M16 | 8 × M30 | 18 × M24 |
| Bolt tension [kN] | 20 | 30 | 60 |
| Temperature [°C] | 340 | 345 | 525 |
| Pressure [MPa] | 0.9 | 18.8 | 9.8 |
| Traction [N] | 1,000 | 3,025 | 1,930 |
| Bending [N·m] | 325 | 2,975 | 7,175 |
| Torsion [N·m] | 100 | 850 | 2,685 |
| Leak rate criterion | `bubble tight' | `bubble tight' | `helium tight' |



Simulations using ABAQUS[®]



Assessment of a bolted-flange connection for ITER Test Blanket Module

T. Zaouter¹, K. Vulliez¹, B. Michel¹, J.-P. Martins² 1. CEA, DES, ISEC, DE2D, SEAD, Laboratoire d'Étanchéité, Univ. Montpellier, Marcoule, France. 2. ITER Organization, Route de Vinon-sur-Verdon, 13067 Saint-Paul-lez-Durance, France.

Temperature below 400 °C: industrial flange NF-EN 1759 Above 400 °C: creep effects \rightarrow custom, water cooled flanges Flanges are rotatable and thermally insulated.



c. helium

Fig. 2: Simplified half view of the flanges For water and lithium-lead assembly: • Static, linear elastic analysis, constant temperature • Mat. prop. from RCC-MRx and ASME-BPVC

• Seals behaviours obtained from fine-scale FEM simulations







e [mm] *Fig. 3: Load-compression curve of the different seals*

Boundary conditions and loadings:

- 1. One face of pipe blocked (except radial disp.), bolt loads
- Ext. mechanical loads (traction, bending, torsion)
- 3. Pipe and channels pressure, temperature and convection

For helium assembly: • Static, linear elastic, fully coupled thermomech. analysis

Fig. 4: Mesh of the helium pipe Thermal contact conductance: global steel-steel: values from lit. \circ silver seal-steel: 15×10³ W/m²·K (exp. data) Convection coefficient: $h_c = \text{Nu} \frac{n_f}{D}$, $\text{Nu} = 0.023 \text{ Re}^{0.8} \text{Pr}^{0.333}$ o gaseous helium pipe, assuming Q = 1.4 kg/s at 525 °C $h_c^0 = 1,115 \text{ W/m}^2 \cdot \text{K}$ • water cooling channels, assuming v = 2 m/s at 34 °C and 1 MPa

 $h_c^1 = 9,280 \text{ W/m}^2 \cdot \text{K}$

Fig. 5: Application of the BC and loadings

Loads



 $\sigma_{
m max}$ [MPa] $\sigma_{
m bolt}$ [MPa] $\sigma_{
m pipe}$ [MPa] T_{\max} [°C] T_{bolt} [°C] $T_{\text{seal}} [^{\circ}C]$ T_{channel} [°C







31st Symposium On Fusion Technology

Blocked face

(except radial disp.

Results

Water & Lithium-Lead flanges

- Maximum equivalent stress at the socket, close to the groove • SS-316L: $R_{p0.2} = 101$ MPa at 340 °C • Eurofer: $R_{p0,2} = 423$ MPa at 340 °C • B21 Class 1: $R_{p0,2} = 880$ MPa at 340 °C
- Important (elastic) stresses
- accommodated with a small amount of plastic strain



Fig. 6: Tresca equivalent stress (in MPa) in the flanges

| | Lithium-Lead | Water | Helium (2 cool. ch.) | Helium (1 cool. ch.) | Tab. 2: Results at particular locations |
|---|--------------|-------|-------------------------|-------------------------|-----------------------------------------|
| | 250 | 276 | 1,340 | 1,097 | the assemblies |
| | 212 | 81 | 383 | 521 | |
| | 107 | 109 | 202 | 203 | |
| | 340 | 345 | 525 | 525 | |
| | — | | 221 | 361 | |
| | | | 64 | 417 | |
|] | | | 62 | 417 | |

Helium flange

- Important stresses induced by the large thermal gradient. • maximal around the socket
- less important if one cooling is inactivated
- plastic permanent marking
- Reasonable stress in the fasteners with two cooling channels

Temperature remains low in the bolts and the seals with two cooling channels, allowing potential thermal design margins.

Temperature reaches unreasonable levels if a single cooling channel is used. • Two cooling channels are necessary

Cooling heat flux of 3.5 kW, a rather high value but within the design acceptance margins.

the cooling power:

Harmonic dependence of φ on Bi₁ \rightarrow difficult to decrease φ Geometrical optimisation of the channels remains a possibility to reduce the cooling power.

| <i>h</i> ⁰ _c [W·m⁻²·K⁻¹] | <i>h</i> ¹ _c [W⋅m ⁻² ⋅K ⁻¹] | <i>φ</i> [kW] | | |
|---------------------------------------------------|-----------------------------------------------------------------------------|------------------|--|--|
| 1,115 | 9,280 | 3.53 o | | |
| 1,115 | 3,061 | 3.37 o | | |
| 1,115 | 845 | 2.91 o | | |
| 1,115 | 382 | 2.39 o | | |
| 6,115 | 9,280 | 4.31 🗆 | | |
| 3,115 | 9,280 | 4.09 🗆 | | |
| 350 | 382 | 1.90 🗆 | | |
| Tab. 3: Coolina heat fluxes | | | | |

To comply with ALARA approach, the studies continue considering design of flange mounted with Quick Disconnecting System (QDS) to replace the fasteners.

This project was conducted under the `SSA-51' working agreement between the CEA and ITER Organization.



Monitoring of the effect of water velocity (through h_c^1) on

 \circ still important even for low h_c^1 due to high convection on the gaseous helium side

can be explained by the simple following model

$$\frac{\varphi}{\rho_{\infty}} = \frac{BI}{1 + Bi}$$

$$\frac{1}{\text{Bi}} = \frac{R_1 - R_0}{\ln R_1 - \ln R_0} \left(\frac{1}{R_0 \text{Bi}_0} + \frac{1}{R_1 \text{Bi}_1} \right)$$
$$Bi_j = h_c^j (R_1 - R_0) / \lambda_s, \qquad j = 0, 1$$



Conclusions

 Implementation of industrial standards for water and lithium-lead bolted flanges.

• Custom design for helium pipe with water cooled flanges.

 FEM thermomechanical analysis demonstrate the feasibility of flange connections.



Acknowledgements