

#### Cross diagnostics measurements of heat load profiles on the lower tungsten ivertor of WEST in L-mode experiments

Nicolas Fedorczak, J Gaspar, Y. Corre, A Grosjean, X. Courtois, J.-P. Gunn, R. Mitteau, R. Dejarnac, J. Bucalossi, E. Tsitrone, et al.

#### ▶ To cite this version:

Nicolas Fedorczak, J Gaspar, Y. Corre, A Grosjean, X. Courtois, et al.. Cross diagnostics measurements of heat load profiles on the lower tungsten ivertor of WEST in L-mode experiments. PSI 24 -24th International Conference on Plasma Surface Interaction in Fusion Devices, The National Fusion Research Institute (NFRI); Korea Nuclear Society, Jan 2021, Daejong, South Korea. cea-03176587

#### HAL Id: cea-03176587 https://cea.hal.science/cea-03176587

Submitted on 22 Mar 2021

**HAL** is a multi-disciplinary open access archive for the deposit and dissemination of scientific research documents, whether they are published or not. The documents may come from teaching and research institutions in France or abroad, or from public or private research centers. L'archive ouverte pluridisciplinaire **HAL**, est destinée au dépôt et à la diffusion de documents scientifiques de niveau recherche, publiés ou non, émanant des établissements d'enseignement et de recherche français ou étrangers, des laboratoires publics ou privés.

#### PSI 2020 Poster session 2 n°55 TP2-063(F)

### EUROfusion WP-PFC Cross diagnostics measurements of heat load profiles on the lower tungsten divertor of WEST in L-mode experiments

- **Nicolas Fedorczak\***, J. Gaspar, Y. Corre, A. Grosjean, X. Courtois, J.P. Gunn, R. Mitteau, R. Dejarnac, J. Bucalossi, E. Tsitrone, T. Ioarer, S. Brezinsek, the WEST team
- \*nicolas.fedorczak@cea.fr CEA, IRFM, F-13108 Saint-Paul-Lez-Durance, France







This work has been camed cut within the framework of the EUROfusion Consortium and has received funding from the European Union's Horizon 2020 research and Innovation programme under grant agreement number 633053. The views and opinions expressed herein do not necessarily reflect these of the European Commission.





CQD

**Summary** 



#### Lower divertor infra-red thermography



Inside of WEST tokamak (with the articulated inspection arm)



### Thermal sensors embedded a few mm from tile surface allow inversion of deposited heat flux



#### **Fiber Bragg Gratings: 12.5mm spatial resolution** [Y. Corre FED 2018]



#### Thermocouples: 37.5 mm spatial resolution

[J. Gaspar JoP 2016]

Cea



- Absolute bulk temperature measurements
- Consistency between TC & FBG
- Data inverted into deposited heat flux profiles:

#### [J. Gaspar NME 2017]

• assuming a "Wagner" shape

$$q(s) = \frac{q_0}{2} \cdot \exp\left(\left(\frac{S^t}{2\lambda_q^t}\right)^2 - \frac{s - s_0}{\lambda_q^t}\right) \cdot \operatorname{erfc}\left(\frac{S^t}{2\lambda_q^t} - \frac{s - s_0}{S^t}\right) + q_{bg}$$

- $s_0, \lambda_q^t \& S^t$  constant in time
- $q_0 \& q_{bg}$  time dependant



PSI 2020 Poster session 2 n°55 | Cross diagnostics measurement of heat load profiles on the lower divertor of WEST in L-mode experiments | N Fedorczak & the WEST team

#### **Ceal** Infra-red thermography of tiles equipped with thermocouples



5

- Absolutely calibrated infra-red endoscopes
  (top ports) looking at lower divertor
  [X. Courtois FED 2019]
- Effective Black body T° profiles extracted along tiles equipped with thermocouples (for emissivity calibration)
- Spatial resolution  $\approx 2.7$  mm

Figure 1: infra-red view of a section of the lower divertor prior to a plasma pulse, following a session of divertor loading. The colormap represents the calibrated effective black body temperature. The pattern exhibits a toroidal modulation (along  $\Phi$ ) due to plasma loading modulated by magnetic ripple, and radial modulations (along R) consequent from varying surface emissivity. Light blue rectangles delimit tiles that are equipped with thermocouples, and along which treatments are performed. Tiles equipped with 4 thermocouples sits at toroidal locations where magnetic ripple produces the maximum deposited heat flux.



WEST 55070 IR endoscope DIVQ6B LOW

Surface emissivity profiles estimated from pre-pulse conditions → show strong inhomogeneity & low values around strike points

Insitu methodology to estimate emissivity profiles from IR & embedded TC:

- Luminance model  $L_{planck}(T_{IR}) = \epsilon L_{planck}(T_{true}) + (1 \epsilon)\epsilon_{amb}L_{planck}(T_{wall})$
- In the pre-pulse phase: tile temperature is uniform and different than wall temperature
- True tile temperature measured by thermocouples
- From N consecutive pre-pulse phases (T increments)

ightarrow Best fit of emissivity values from IR ad TC data

#### Important Results:

- Emissivity varies strongly across the divertor surface
- Very low ( $\epsilon pprox 0.1$ ) around the main outer strike point
- Varies across campaigns (possibly related to deposits)

#### [J. Gaspar this conference Poster session 3 n°66]



6



#### CCC Corrected IR data inverted with TEDDY (equivalent to THEODOR)



- Thermal inversion with TEDDY [N. Fedorczak Phys.Script. 2020]
- Outer target profiles well fitted with a "Wagner" shape.
- Fit less robust for the inner target (multiple peaks related to ExB flows?)





Heat flux pattern toroidally modulated by the incidence angle (magnetic ripple)



8

#### Finite magnetic ripple in WEST (~1%)

 $\rightarrow$  The magnetic field incidence angle on the divertor surface is toroidally modulated

### Toroidal modulations of IR estimated heat flux at outer target consistent with:

- geometrical projection of a toroidally uniform parallel heat flux
- plus a toroidally uniform background\* much weaker than peak heat flux (one order of magnitude)
- (background value ~2 times larger than radiative flux estimated from bolometry inversion)

### $\rightarrow$ Validates the common assumption that deposited heat flux is dominated by the projection of a parallel heat flux



#### toroidal modulation of deposited heat flux



#### IR & FBG estimated heat flux profiles agree reasonably well



- Heat flux profiles estimated at the maximum ripple modulation
- Good agreement on peak amplitude and position, less on profile width



#### IR & FBG agree well on estimated peak and integrated heat flux



- Data base of more than 100 L-mode discharges with stable (>4s) heating phases
- Consider only outer target (inner profile shape not understood)
- Local peak heat flux well matched by the 2 diagnostics (20% scatter)
- Integrated tile energy also well matched ( $E \equiv \int \int q_{\perp} ds dt$  integral along tile length & time)





### IR & FBG estimated target heat flux width $\lambda_q^t$ vary linearly with magnetic flux expansion



11

- Consider stable L-mode heated plasma scenario
- Magnetic flux expansion at target  $(f_X)$  varied by changing the X-point height wrt divertor surface.
- Outer target heat flux width  $\lambda_q^t$  estimated by "Wagner" fit of experimental profiles from IR & FBG
- Good linearity of  $\lambda_q^t$  function of  $f_X$ 
  - → Validates the common normalization  $\lambda_q \equiv \lambda_q^t / f_X$

#### Important observations:

- $\lambda_q$  from IR is about 3 times lower than from FBG in scenarios  $\lambda_q^{IR} \approx 2.8$ mm &  $\lambda_q^{FBG} \approx 7.4$ mm
- For this WEST scenario, multi-machine scaling laws predict  $\lambda_q \approx 2.9 - 3.7 \text{ mm}$  L-mode database [A. Scarabosio JNM 2013]  $\lambda_q \approx 1.1 - 3.7 \text{ mm}$  H-mode database [T. Eich NF 2013]





## IR & FBG estimated target heat flux width $\lambda_q^t$ indeed disagree on large database



- Data base of more than 100 L-mode discharges with stable (>4s) heating phases
- Strong dispersion found in the comparison between  $\lambda_q^t$  from IR & FBG
- Statistically  $\lambda_q^{FBG} \approx 3\lambda_q^{IR}$

C 2 2

#### But peak heat flux and integral are well matched

→ Different profile widths compensated by different background heat fluxes from IR & FBG

# → This could suggest a systematic error in the treatment of IR reflections (self-reflections) Still under investigation







13

- Heat flux deposited profiles routinely measured by embedded thermal sensors (FBG & TC), IR thermography and flush mounted Langmuir probes
- WEST experiments validate commonly accepted geometrical assumptions:
  - Target heat flux width varies linearly with magnetic flux expansion, allowing to define a remapped channel heat flux width  $\lambda_q \equiv \lambda_q^t / f_X$
  - Toroidal deposited heat flux modulation is consistent with the magnetic projection of a parallel heat flux + homogeneous background (radiation & neutrals)

#### $\rightarrow$ Key steps to prepare WEST scenarios for ITER grade PFU testing

- FBG & IR estimated peak heat flux and flux integral agree quantitatively
- Heat flux channel widths from IR are about 3X lower than estimated from FBG
  → This could suggest an inappropriate treatment of IR reflections in metallic wall
- Comparison with Langmuir probes ongoing