



## Experiments on interactions of molten steel with suboxidized corium crust for in-vessel melt retention

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# In-Vessel Retention: Molten stainless steel - corium crust interaction

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## Abstract

In the framework of the European IVMR project, dedicated to the assessment of In-Vessel Retention (IVR) strategy for high power reactors, VITI-CORMET tests, performed in VITI facility (CEA-Cadarache/PLINIUS Severe Accident Platform), intend to study the interaction between a molten 304L stainless steel droplet and suboxidized solid corium crust, referred to as C-70 ( $(\text{U}_{0.54}, \text{Zr}_{0.46})\text{O}_{1.74}$ ). In the present paper, original experimental results on molten steel penetration through a corium crust are presented. Identification of penetration mechanisms is performed by SEM/EDX analyses, and has shown that molten steel can penetrate through pre-existing cracks or by a dissolution process. The latter is found to be the dominant penetration mechanism and involves material transport from the crust to molten steel. Penetration kinetics is studied by measuring the average penetration length at different exposure times. It is found that the penetration by dissolution decelerates with time, while U and Zr contents tend to reach an equilibrium value within the metallic droplet.

**Keywords:** In-Vessel Retention, Focusing effect, Molten stainless steel, Suboxidized solid corium

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## 1. Introduction

During a severe accident in a Light Water Reactor (LWR), core meltdown may result in the formation of a molten corium pool in the lower head of the reactor vessel. The IVR as a severe accident management strategy aims to contain and stabilize the corium within the reactor pressure vessel by external cooling of the vessel wall using natural or forced water convection. This strategy is highly desirable since it would minimize the risks of containment failure and has already been adapted for some low power LWRs (e.g: Loviisa). The success of IVR strategy mainly depends the heat flux transferred by the corium pool to the vessel wall, which must remains lower than the Critical Heat Flux (CHF) at all points around the lower head. Depending on the reactor characteristics, the CHF ranges from  $1.5 \text{ MW.m}^{-2}$  to  $2 \text{ MW.m}^{-2}$  [1, 2]. One of the phenomena limiting the IVR strategy is the so-called "Focusing Effect" due to the formation of a "thin" light metal layer on top of the corium pool in direct contact with the vessel wall. Concentration of the heat flux by focusing effect can lead to the vessel failure due to high heat flux which can be greater than the CHF. In general, the intensity of the focusing effect increases when the thickness of the upper metallic layer decreases [3, 4, 5].

The thickness of top metal layer can vary according to the stratification mechanism. This physical process has been previously observed in OECD MASCA program [6], in which the interaction between molten Stainless Steel (SS) and molten suboxidized  $\text{UO}_2+\text{ZrO}_2+\text{Zr}$  corium in the miscibility gap has been studied. It has been established that material exchange occurs between these two immiscible phases, leading to an enrichment of the liquid steel in both uranium and zirconium. According to the system composition, the metallic liquid (SS + U + Zr) density can be either higher or lower than the oxidic liquid density. Depending on its density, the metallic liquid can be located at the bottom or on top of the corium pool when equilibrium is reached.

During the transient process of stratification, it has been shown that the liquid phases configuration of the corium pool can strongly affect the IVR strat-

egy chances of success [7, 8], in particular because of the possible thinning of top  
 metal layer. At the transitional stage of stratification, a part of SS is relocated  
 at the bottom and the remaining SS lays on top, forming the upper metallic  
 layer. In this case, a suboxidized solid corium crust can separate the top metal  
 layer from the oxidic liquid, and the 3-layer molten pool is established. Within  
 the 3-layer configuration the focusing effect is higher. A small scale experimen-  
 tal study of the impact of a corium crust on the stratification kinetics has been  
 carried out in the scope of the CORDEB program [9]. It has been shown that  
 the corium crust secured the long-term existence of the 3-layer corium pool con-  
 figuration compared to the fast stratification observed in MASCA experiments  
 [6] where no crust has been observed on top of the corium pool. The stratifi-  
 cation kinetics is limited by the mass transfer of the upper metal layer through  
 the "permeable" oxidic crust. The penetration of the molten metal into the  
 crust can occur according to two main mechanisms that can be coupled: disso-  
 lution mechanism, propagation through cracks mechanism. However, it seems  
 difficult to highlight the phenomenology of the interaction between molten steel  
 and corium crust from CORDEB experiments because strong thermal gradients  
 are present at the crust location and the physical system corium crust-molten  
 steel can not be decoupled from the liquid pool. Until now, there is a lack of  
 knowledge on this interaction and on the first-order parameters governing the  
 penetration kinetics.

In the present paper, an experimental study, called VITI-CORMET, of the  
 interaction between liquid 304L stainless steel and suboxidized corium crust in  
 quasi-isothermal condition is presented. This study is a focus on the physical  
 system corium crust-molten steel. In order to carry out this analysis, seven  
 experiments have been performed at CEA-Cadarache in the VITI facility of  
 the PLINIUS platform. The 304L stainless steel, corresponding to the internal  
 steel structures in French nuclear reactor vessel lower-head, is melted on a sub-  
 oxidized corium crust having a composition  $(U_{0.54}, Zr_{0.46})O_{1.74}$ , the so-called  
 C-70. The experimental grid has been built on two key parameters involved in  
 these mechanisms: the temperature, ranging from 1600 °C to 1800 °C, and the



exposure time, varying from 30 min to 180 min.

A description of the VITI facility, the experimental procedures of the corium crust synthesis (VITI-CREACOR) and VITI-CORMET experiments, and post-  
65 test analyses procedures are detailed in Section 2. Original results on penetration mechanisms and penetration kinetics of the molten steel through the crust are presented in Section 3. A discussion on these new insights are given in Section 4. Finally, a conclusion and some prospects are presented in Section 5.

## 70 2. Experimental procedure

### *2.1. Apparatus VITI-CREACOR/CORMET*

The layout of VITI facility is displayed in Figure 1 for a VITI-CREACOR configuration. The charge (1) is placed in a crucible (2) (41 mm diameter - 48 mm height) made of tungsten. The whole system is heated by direct induction method : the water-cooled inductor (4), connected to the power generator  
75 (8) at working frequency  $f = 180$  kHz, is electromagnetically coupled with the electroconductive crucible (2). The latter heats the charge up to the target temperature, mainly by conduction. There is a thermal shield (3) around the crucible in order to limit thermal losses. The charge crucible and the thermal  
80 shield sits upon a dense graphite support (6).

The VITI confinement vessel (7) is water-cooled and allows to work in a controlled atmosphere. The ambient atmosphere is composed of inert argon gas at absolute pressure 1.3-1.5 bar measured with the manometer (10). The initial atmosphere composed of ambient air is removed by several vacuum pump-  
85 ing/Argon sweeping cycles. Two bi-chromatic video pyrometers (9), with working wavelengths  $\lambda_1 = 0.95 \mu\text{m}$  and  $\lambda_2 = 1.05 \mu\text{m}$ , allow temperature measurements and experimental imaging. One pyrometer is focused on the charge surface through the upper window (5), while the second one is focused on the side of the crucible. The focus point of the second pyrometer is located just beneath  
90 the height of the melt after complete melting, in order to measure the tempera-

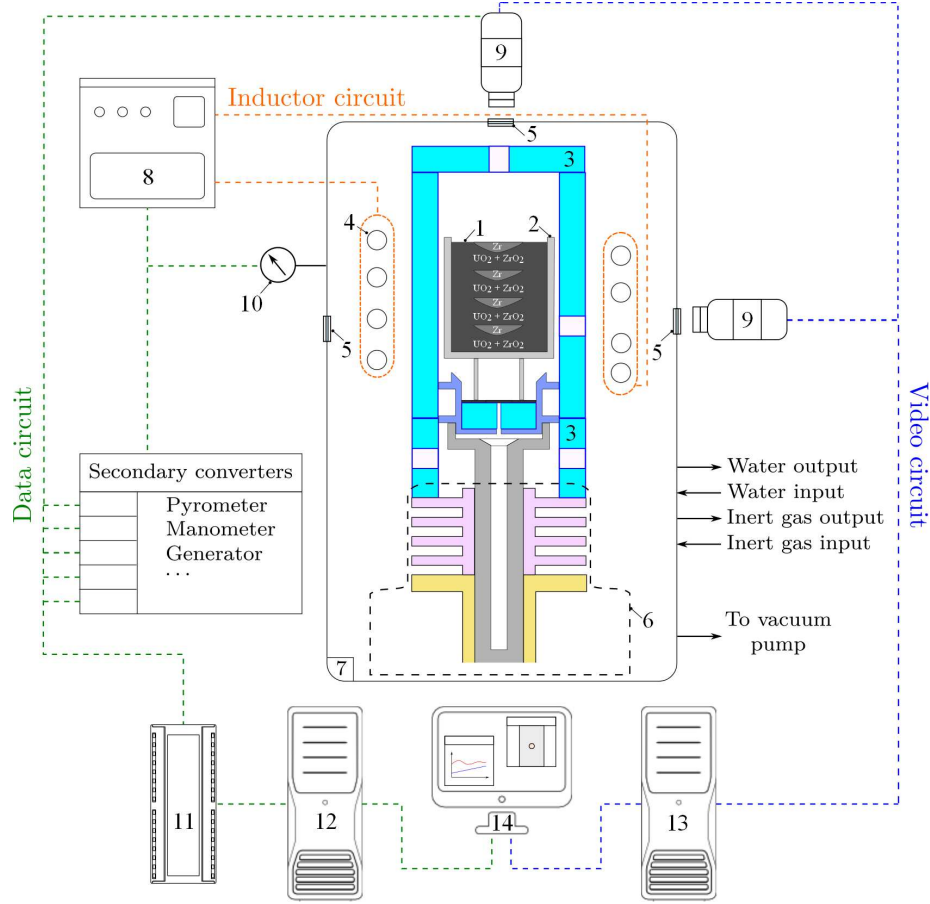


Figure 1: Scheme of the VITI facility in VITI-CREACOR configuration. 1: powder mixture; 2: tungsten crucible; 3: thermal shield; 4: inductor; 5: windows; 6: crucible support system; 7: confinement vessel; 8: generator; 9: video-pyrometer; 10: manometer; 11: data bus; 12: data acquisition computer; 13: imaging computer; 14: monitoring and data treatment.

ture of the crucible in contact with the melt. Data are collected by the data bus (11) and are sent to the data acquisition computer (12). The imaging computer (13) is used for sample monitoring, and allows real time and post-test analyses.

## 2.2. Corium crust formation and characterization: VITI-CREACOR

95 The charge is composed of a mixture of  $\text{UO}_2$  (grain size  $\leq 100 \mu\text{m}$ ) and  $\text{ZrO}_2$  (high purity CEZUS<sup>®</sup> zirconia, 99.95 % purity, grain size  $\leq 2 \mu\text{m}$ ) powders

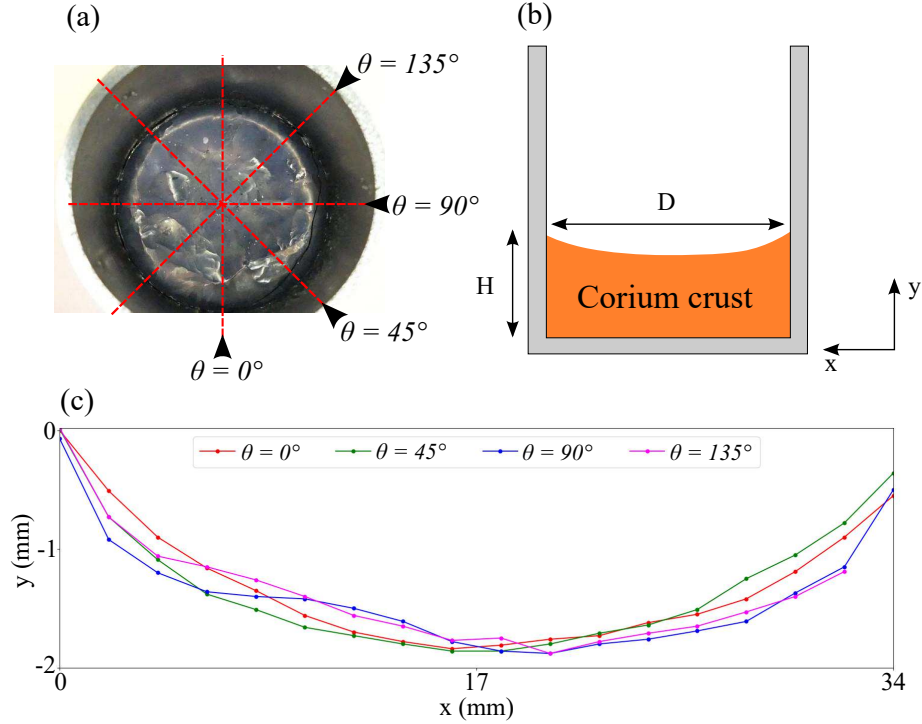


Figure 2: Surface characterization of the corium crust. (a) Corium crust after VITI-CREACOR, dashed red lines represents the four concavity measurement diameters, (b) schematic cross section of the crust with dimensions and axis system, (c) depth variation of the crust meniscus ( $y$ ) with respect to the length of the four measurement diameters ( $x$ )

ground in a mortar and Zr metal granulates (grain size  $\leq 6$  mm). The mass of the charge is equal to 155 g, and it is placed into the crucible in order to obtain a target corium composition  $(U_{0.54}Zr_{0.46})O_{1.74}$ , a so-called C-70 corium, with an atomic ratio  $U/Zr = 1.2$ . Zirconium granulates are deposited in layers within the powder mixture in order to avoid contact with the crucible during the melting process. The charge is heated up to the target temperature, upper than the liquidus temperature of the mixture calculated with Thermo-Calc Software [10] and NUCLEA database [11, 12], ensuring liquid state of the mixture ( $T_{fus} = 2520$  °C). The charge is held in the liquid state for about ten minutes in order to reach a stationary condition, then the molten charge is rapidly cooled

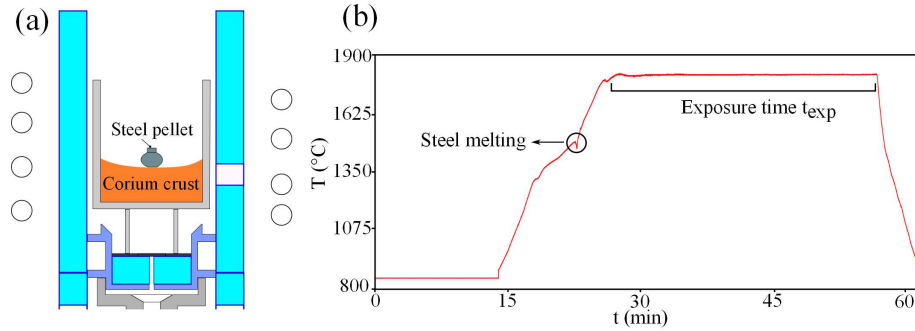


Figure 3: (a) Schematic view of the VITI-CORMET test section. (b) Typical thermograph of VITI-CORMET experiments

by turning off the inductor.

After the corium formation step, the suboxidized corium crust is extracted from the confinement vessel in order to characterize its surface. A typical top view of a corium crust is displayed in Figure 2a. The corium crusts obtained by VITI-CREACOR have a diameter  $D$  equal to 41 mm and a maximum height  $H$  of about 17 mm, these dimensions are schematically represented in Figure 2b. The surface of the crusts is generally concave. A depth gauge is used to measure the depth of the meniscus, along four different diameters, every two millimeters (see Figure 2a). Figure 2c shows the depth variation of the crust as a function of the distance along the diameters. The surface is quite symmetric and has a maximum relative depth of about 1.8 mm.

These experimental data are necessary to evaluate the interaction between stainless steel and corium crust (VITI-CORMET), and the penetration length of the liquid steel through the corium crust. The data represented in Figure 2c are used as initial position of the molten steel bottom surface.

### 2.3. Molten steel-suboxidized corium crust interaction: VITI-CORMET

A schematic view of the VITI-CORMET configuration is shown in Figure 3a. A 304L stainless steel pellet, with a mass of about 2 g, lays on the corium crust, synthesized during VITI-CREACOR stage, and heated up to the target temperature with a slope of about  $1.8\text{ }^{\circ}\text{C}\cdot\text{s}^{-1}$  using the same protocol as the

one described in Section 2.1. The ambient atmosphere is composed of inert argon at absolute pressure 1.3-1.5 bar. A typical thermograph is shown in Figure 3b, where the 304L steel pellet melting is detected at 1460 °C by a temperature drop. Then, the quasi-isotherm condition is rapidly reached. In general the 304L melting point is observed with a high reproducibility from one experiment to another with a value of about 1450 °C. This experimental value is in good agreement with melting temperature of 304L stainless steel reported in the literature [5, 13], which confirms the good reliability of the measurement. One can observe the high stability of the temperature during the exposure time  $t_{exp}$ . Finally, the molten droplet is rapidly cooled by turning off the inductor.

#### 2.4. Experimental grid and characterization processes

In order to study the interaction between 304L molten steel and suboxidized corium crust in quasi-isotherm condition, seven VITI-CORMET tests are performed with two setting parameters: temperature and exposure time. This experimental grid is conducted with a temperature range from 1600 °C to 1800 °C and an exposure time ranging from 30 min to 180 min. The target corium composition ( $U_{0.54}Zr_{0.46}$ ) $O_{1.74}$  is the same in all the experiments, as well as its mass (155 g). In these seven experiments, 304L stainless steel pellets, with a mass of about 2 g, are melted on the corium crust. Finally, the corium crust CREACOR-1 is used as reference to compare the targeted corium composition with the experimental one, since this sample did not interact with the molten steel.

Post-test analyses are performed using Scanning Electron Microscopy (SEM) imaging. For SEM observations, samples are cut along a concavity measurement diameter displayed in Figure 2a, mounted in Struers-EpoFix resin and polished with an automatic polishing machine using SiC polishing disks and then diamond solution. Carbon deposits of about 30 nm are made in order to metallize the samples. Samples are examined with a ZEISS EVO HD15-MA Scanning Electron Microscope in order to analyse penetration mechanisms of the molten steel through the crust as well as penetration kinetics and material transport

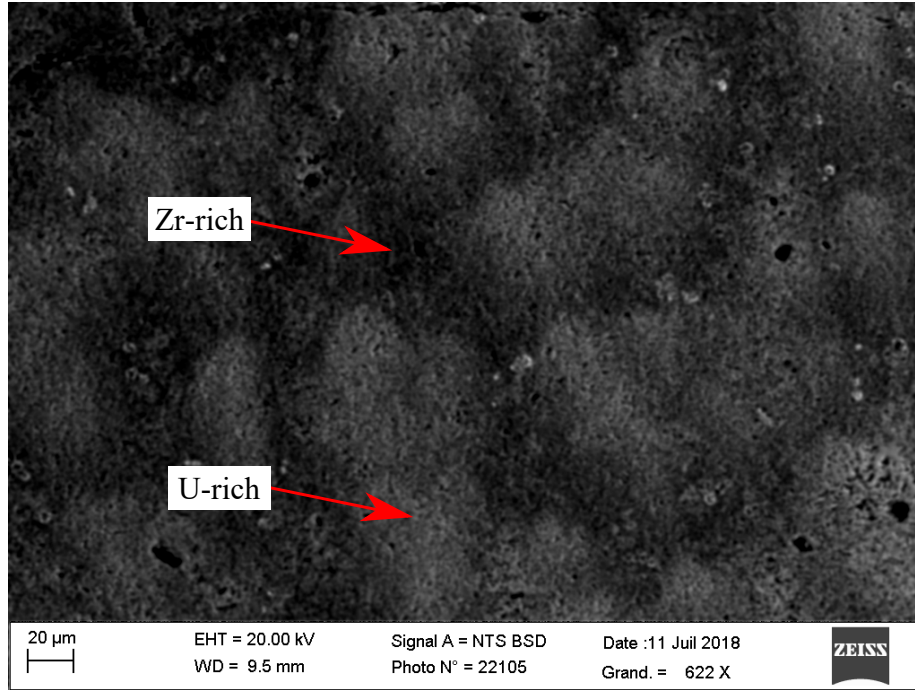


Figure 4: BSE image of the crust from the reference sample CREACOR-1. Image obtained with low brightness and high contrast.

occurring in this system. Oxygen measurement for local corium analyses remains very challenging due to large uncertainties. Nevertheless, some insightful features can be highlighted for experimental interpretation.

### 160 3. Experimental results

#### 3.1. Analysis of the reference sample CREACOR-1

In order to compare the targeted corium crust composition ( $\text{U}_{0.54}\text{Zr}_{0.46}\text{O}_{1.74}$ ) with the experimental one, a corium crust that did not interact with the molten steel is analysed (sample CREACOR-1). A BackScattered Electrons (BSE) image of the crust from CREACOR-1 is displayed on Figure 4. This image is  
165 taken with low brightness and high contrast to distinguish the heterogeneities of the composition. Two kinds of regions can be observed in the crust : U-rich

Windows	U at.%	Zr at.%	O at.%	U/Zr	Formula
Global	20.3	16	63.7	1.27	(U <sub>0.56</sub> Zr <sub>0.44</sub> )O <sub>1.75</sub>
U-rich	21.1	13.8	65.1	1.53	(U <sub>0.60</sub> Zr <sub>0.40</sub> )O <sub>1.86</sub>
Zr-rich	19.4	17.6	63	1.10	(U <sub>0.52</sub> Zr <sub>0.48</sub> )O <sub>1.70</sub>

Table 1: Average composition of the corium crust for the reference sample CREACOR-1.

(brighter areas) and Zr-rich (darker areas) regions. The average composition of the crust is measured using windows including both U-rich and Zr-rich regions (called "Global" windows) and windows dedicated to either U-rich regions or Zr-rich regions. The results are summarized in Table 1 for these three types of windows and the molecular formulations are presented to compare them to the target corium composition (U<sub>0.54</sub>Zr<sub>0.46</sub>)O<sub>1.74</sub>. The measured global composition is closed to the targeted one, but a slight difference is observed on the atomic ratio U/Zr which is equal to 1.27 with respect to the expected one (1.2). Larger deviations from the target composition are observed regarding both U-rich and Zr-rich regions. The U-rich regions are characterized by an atomic ratio U/Zr equal to 1.53 and appear to be more oxidized than the Zr-rich regions, the latter being characterized by an atomic ratio U/Zr equal to 1.10.

### 3.2. Penetration mechanisms

A BSE composite image of sample cross section for 304L/(U<sub>0.54</sub>Zr<sub>0.46</sub>)O<sub>1.74</sub> system cooled from 1700 °C after 180 min exposure time is displayed on Figure 5. This micrograph is representative of the other samples obtained from VITICORMET experiments. Three distinct regions can be highlighted in the system. The first region is the solidified metallic droplet whose size varies according to the spreading. The second region is the interaction zone, which is delimited by the red line and defined by the distance between the initial surface of the crust and the deepest metallic inclusions observed in this crust. The third region corresponds to the corium crust that did not interact with the molten steel.

During the interaction, molten steel penetrates through the corium crust. Two penetration mechanisms are identified in these experiments. Firstly, molten steel can penetrate through the crust by a dissolution mechanism. Figure 6a

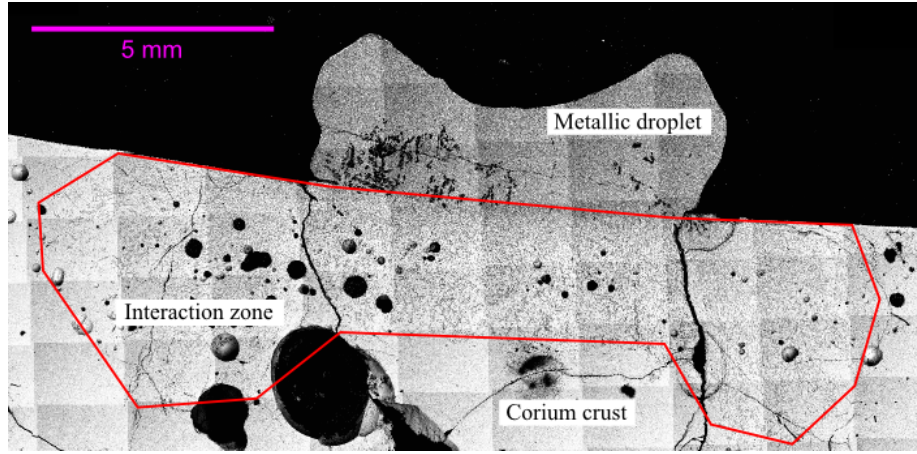


Figure 5: BSE composite image of cross section for 304L/(U<sub>0.54</sub>Zr<sub>0.46</sub>)O<sub>1.74</sub> system cooled after 180 min from 1700 °C. The red line delimits the interaction zone.

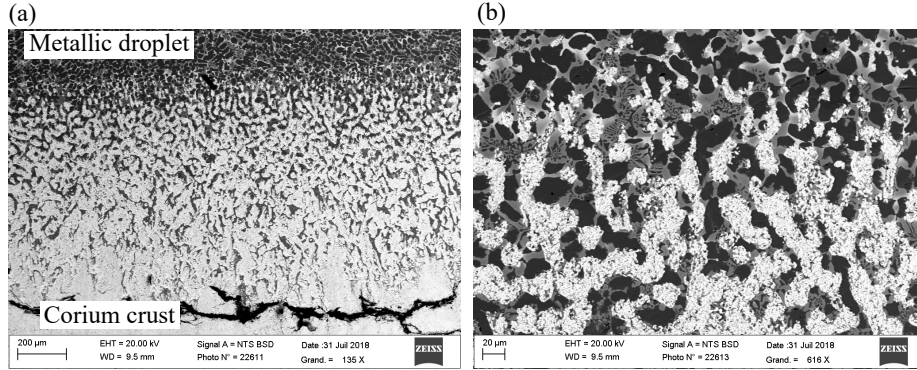


Figure 6: (a) BSE image of the interaction zone formed by dissolution and (b) magnification of the upper part of the interaction zone for 304L/(U<sub>0.54</sub>Zr<sub>0.46</sub>)O<sub>1.74</sub> system cooled after 30 min from 1800 °C.

shows the microstructure resulting from the dissolution process, which represents a part of the interaction zone. The upper part of this region contains discontinuous small islands of corium crust surrounded by metal (displayed in Figure 6b). In the middle part, an interconnected channels structure filled with metal between large discontinuous crust islands is observed. While in the lower part of the interaction zone only metallic inclusions are observed in a continuous corium crust. This morphological sequence indicates that the dissolution



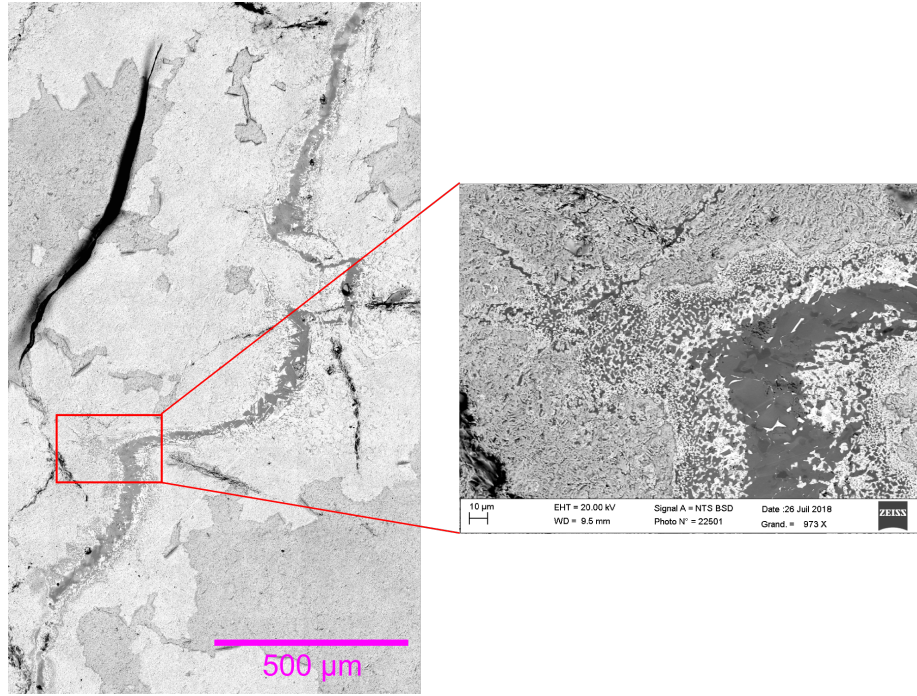


Figure 7: BSE image of a crack filled by metal (left) and magnification of this crack (right) for  $304L/(U_{0.54}Zr_{0.46})O_{1.74}$  system cooled after 180 min from 1800 °C.

process leads to an heterogeneous molten steel penetration through the corium crust and involves the existence of preferential path such as grain boundaries, sub-grain boundaries or porosities.

The other way for the molten steel to penetrate within the corium crust is through macroscopic defects such as constraints which can appear during the solidification process. Formation of cracks is dependent on the solidification process in the CREACOR configuration, as well as in the reactor case. Out of seven experiments, only one crack filled by metal is observed, as displayed on Figure 7 (left panel). The apparent diameter varies between 10  $\mu\text{m}$  and 100  $\mu\text{m}$ . The right panel in Figure 7 represents a magnification of the crack filled by metal and shows that the molten steel interacts with the crust during its penetration through the crack, as U-rich solidified particles (white particles) are observed near the edges where a dissolution process takes place. This attack

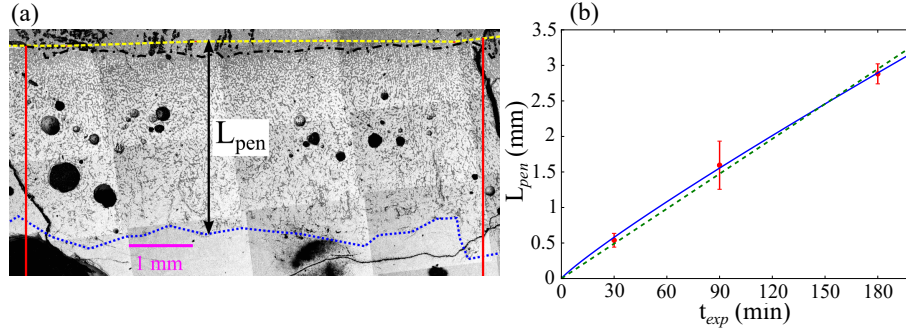


Figure 8: (a) BSE composite image representing the penetration length analyses procedure. (b) Average penetration length of molten steel dissolution attack as function of the exposure time at  $T=1700\text{ }^{\circ}\text{C}$ . Solid line corresponds to the fitting curve using Equation 1. "Dash" line corresponds to a linear fit. Error bars represent the standard deviation.

contributes to an enlargement of the crack. Although penetration through the crack could be a faster way for the metal to penetrate within the crust, it is nevertheless minor compared to dissolution penetration as it strongly depends on the number of cracks connected to the surface of the crust. Let us rather focus on the kinetics of penetration by dissolution attack.

### 3.3. Penetration kinetics at $T=1700\text{ }^{\circ}\text{C}$

The average penetration length ( $L_{pen}$ ) of molten steel through the corium crust is measured using the analysis procedure shown in Figure 8a. The axial distance between the initial surface of the crust ("dash" yellow line) and the line where the deepest metallic inclusions are observed ("dot" blue line) is measured. This measurement is repeated a hundred times between the two triple points of the metallic droplet marked by the red vertical solid lines. In order to visualize the part of the totally dissolved crust, a line representing the location of the last partially dissolved corium grains observed in the upper part of the interaction zone is plotted on the image ("dashdot" black line).

The evolution of the average penetration lengths, for  $T=1700\text{ }^{\circ}\text{C}$ , as a function of the exposure time is displayed in Figure 8b. Experimental data are fitted, assuming a deceleration of the penetration with time, following the general law

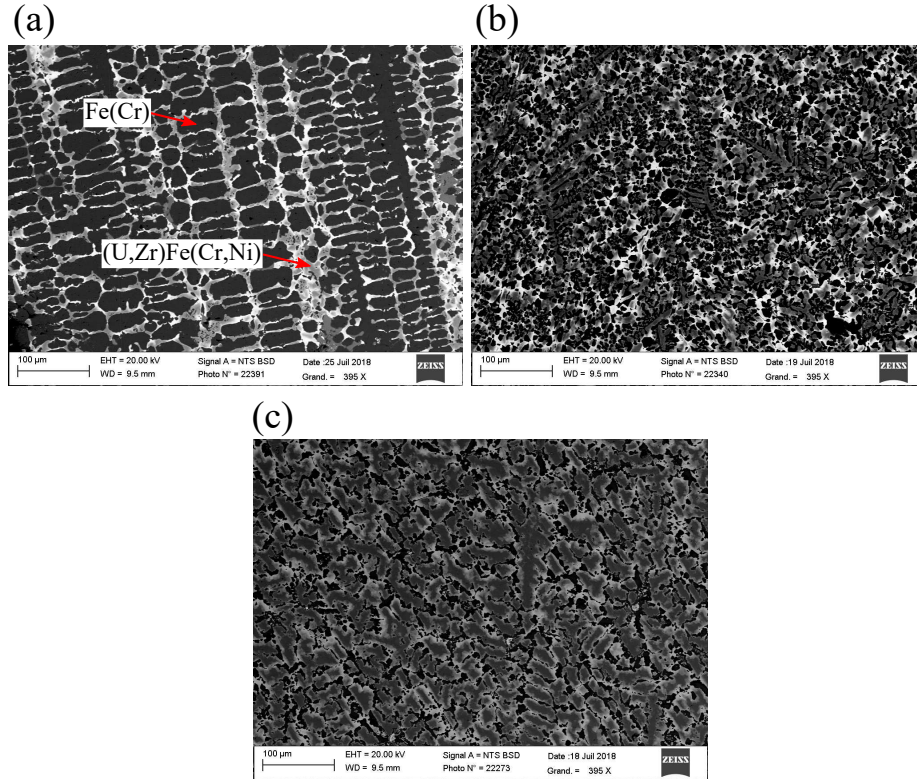


Figure 9: Microstructures of the solidified metallic droplet obtained from VITI-CORMET tests performed at 1700 °C with an exposure time of 30 min (a), 90 min (b) and 180 min (c).

[14] :

$$L_{pen} = A t_{exp}^{\alpha} \quad (1)$$

where A and  $\alpha$  are two constants. A deceleration of the molten steel penetration is observed ( $\alpha = 0.90 \pm 0.04 < 1$ ).

The dissolution mechanism implies material transport from the crust to the  
 235 molten metal leading to the enrichment in uranium and zirconium of the upper  
 metallic droplet as well as the metallic inclusions (interstitial metal). This  
 phenomenon is analysed in the following section.

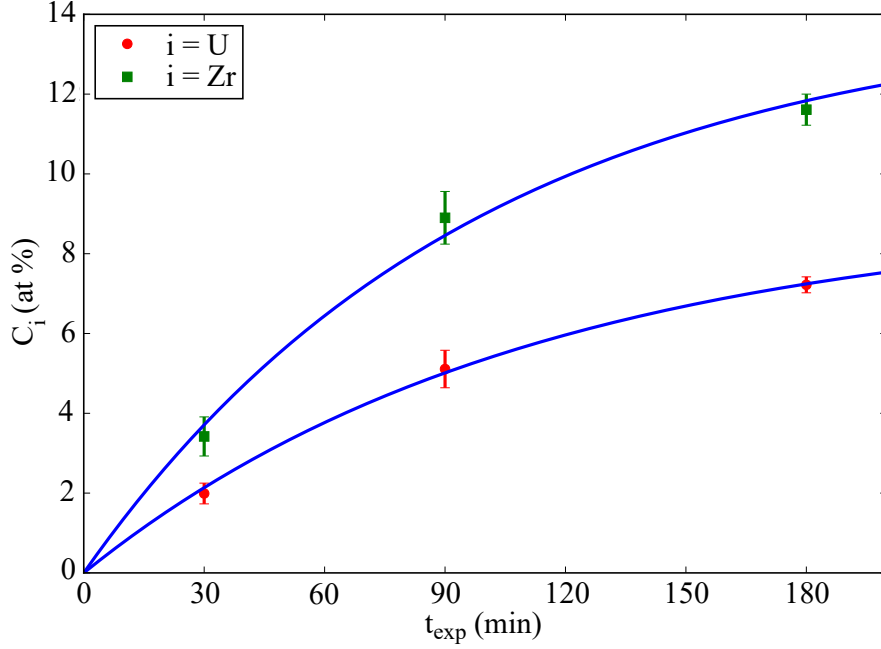


Figure 10: Time evolution of the average concentration in U and Zr of the upper metallic droplet at 1700 °C. Solid lines correspond to the fitting curves.

#### 3.4. Metallic droplet composition at 1700 °C

The microstructures of the solidified metallic droplet for an exposure time of 30, 90 and 180 min are given in Figure 9a, b and c respectively. Two phases are observed in the samples : a dendritic Fe(Cr) phase (black areas) and a continuous solid solution (U,Zr)Fe(Cr,Ni). In the latter, composition gradients are observed with U-rich regions (brighter areas) and Zr-rich regions (darker areas). The proportion of this phase varies with the exposure time, as it is shown in Figure 9. The longer the exposure time is the higher is the proportion of this solid solution is, resulting in a gradual vanishing of the Fe(Cr) phase.

To quantify the enrichment of the upper metallic droplet as function of exposure time, numerous EDX analyses are performed on each sample with large enough windows in order to include a sufficient amount of both Fe(Cr) and (U,Zr)Fe(Cr,Ni) phases. The time evolution of the average concentration in

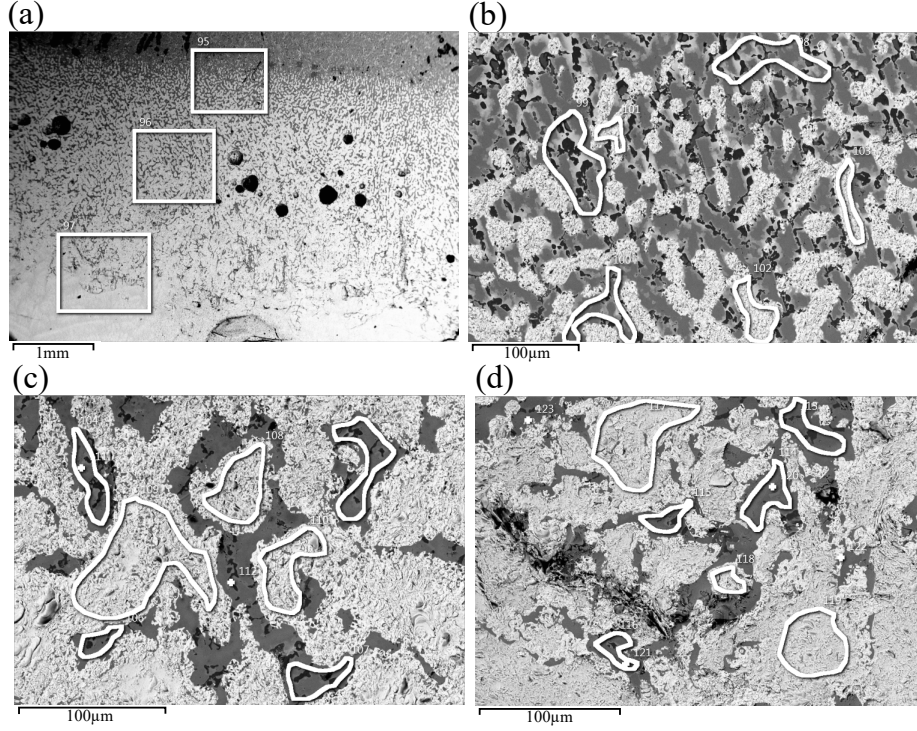


Figure 11: (a) BSE image representing the interaction zone with three areas of interest (white rectangles) to study the interstitial metal composition. Magnification of these areas: (b) top, (c) middle and (d) bottom. Interstitial metal corresponds to dark phases while the brighter phase represents the corium crust. 304L/(U<sub>0.54</sub>Zr<sub>0.46</sub>)O<sub>1.74</sub> system cooled after 180 min from 1700 °C.

uranium and zirconium of the metallic droplet is displayed in Figure 10. Both contents increase with time but the atomic ratio U/Zr stays quasi-constant as its value is equal to 0.58, 0.57 and 0.62 at  $t_{exp} = 30, 90$ , and 180 min respectively. Furthermore, the average concentrations seem to tend toward an equilibrium value and no concentration gradient is observed in the metallic droplet. Experimental results are fitted by a general equation of dissolution kinetics [15, 16] given by:

$$C_i(t) = C_{eq}^i(1 - \exp(-k_i t)) \quad (2)$$

where index  $i$  refers to uranium or zirconium,  $C_{eq}^i$  corresponds to the equilib-

Location	U at. %	Zr at. %	O at. %	Fe at. %	Cr at. %	Ni at. %	U/Zr
Top	6.1	15.9	6.8	51.1	14.6	5.5	0.37
Middle	1.7	18.9	7.6	51	15.1	5.7	0.09
Bottom	1.6	22.4	6.5	49.2	14.8	5.5	0.07

Table 2: Average concentrations of major elements composing the interstitial metal and atomic ratio U/Zr for 304L/(U<sub>0.54</sub>Zr<sub>0.46</sub>)O<sub>1.74</sub> system cooled after 180 min from 1700 °C.

rium concentration of element  $i$  and  $k_i$  is an effective dissolution rate constant of element  $i$ . By analogy with a liquid dissolving a solid particle,  $k_i$  is associated to a diffusion coefficient  $D_i$ , the surface area  $S$ , the volume of the dissolution medium and the thickness of the diffusion layer  $\delta$  by the expression  $k_i = D_i S / V \delta$ . The values obtained from the fit are  $k_U = 1.51 \times 10^{-4} \pm 4 \times 10^{-6} \text{ s}^{-1}$  and  $k_{Zr} = 1.71 \times 10^{-4} \pm 9 \times 10^{-6} \text{ s}^{-1}$ , involving a dissolution rate slightly more important for Zr. Equilibrium concentrations found from the fit are equals to 9 at.% and 14.1 at.% for uranium and zirconium respectively, leading to an atomic ratio U/Zr equals to 0.64, in fair agreement with the experimental values.

### 3.5. Interstitial metal composition at 1700 °C

The situation is quite different regarding the interstitial metal located in the channels of the interaction zone. The average composition of the interstitial metal are measured along the height of the interaction zone. An example of these measurements are shown in Figure 11 for the CORMET-3 sample (T=1700 °C,  $t_{exp}$ =180 min). Figure 11a represents three distinct areas of interest (white rectangle) located on top, at the middle and at the bottom of the interaction zone, while Figure 11b, c and d display a magnification of these areas respectively where composition measurements are performed (white unstructured windows).

Table 2 shows the average compositions of the major elements forming the interstitial metal at these locations. A concentration gradient of U and Zr is observed between the lower part and the upper part of the interaction zone. A maximum (minimum) Zr (U) content is observed at the bottom with 22.4 (1.6) at.%, while a minimum (maximum) Zr (U) content is observed on top with 15.6 (6.1) at.%. This gradient is also well represented by the atomic ratio

U/Zr varying from 0.07 to 0.37 between the extremities of the interaction zone. Regarding the other components (Fe, Cr, Ni and O) their concentrations are quite constant along the height of the interaction zone. It may be noted that this behaviour discussed for CORMET-3 sample is observed in all other tests independently of the experimental conditions.

#### 4. Discussion

In these CORMET experiments, it is found that the liquid 304L stainless steel strongly interacts with the suboxidized corium crust. The micrographic observations made on the longitudinal cross section of the samples have shown that the liquid steel can penetrate through the corium crust using macroscopic defects, such as cracks, or by a dissolution mechanism. The dissolution process leads to an interconnected network of channels filled by molten metal and defining the interaction zone. As observed, the upper part of the interaction zone is composed of discontinuous corium grains partially dissolved and surrounded by metal, while in the lower part metallic inclusion within a continuous corium crust is observed. These observations involve a heterogeneous way of penetration for the molten steel through the crust and consequently existence of preferential paths such as grain boundaries, sub-grain boundaries or porosities. In addition, it is found that the dissolution process lead to material transport changing the nature of the molten steel in a metal enriched in uranium and zirconium. However, concentration gradients are found between the metallic inclusions in the lower part of the interaction zone, having an atomic ratio  $U/Zr \approx 0.07$  (Zr-rich liquid), and the metallic droplet having an atomic ratio  $U/Zr \approx 0.6$ . Between them, an intermediate metal is found on top of the interaction zone and surrounding the discontinuous corium grains. This one has an atomic ratio  $U/Zr \approx 0.4$ . In all cases, the values of the atomic ratio indicate a preferential extraction of zirconium from the corium crust to the liquid steel, but with a gradient according to the front position of the interaction.

These observations bring the assumption that the dissolution mechanism

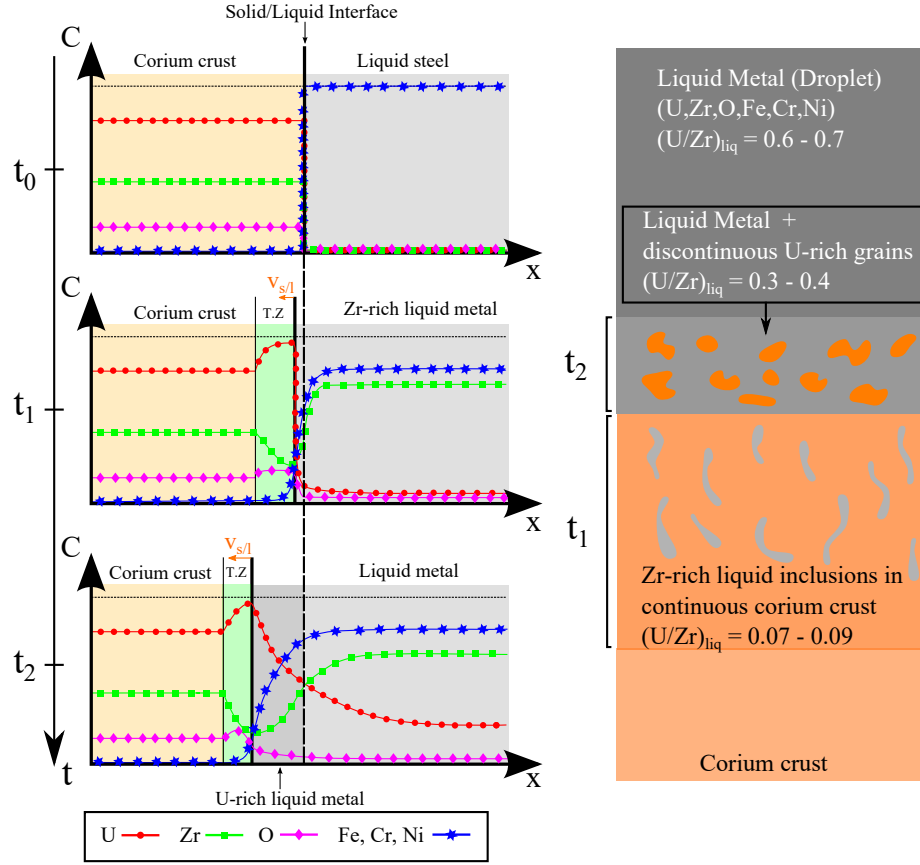


Figure 12: Schematic view of the concentration profiles at the interface corium crust-liquid steel (left part). Timeline is given in order to highlight the two-step dissolution process. Schematic view of the interaction zone (right part) displaying the correlation between spatial observations and the two-step process.

could be a two-step process depending on the interaction time. In Figure 12, a schematic view of the concentration profiles at the interface corium crust-liquid steel is displayed in order to describe the two-step process. For the sake of clarity, the steel elements are drawn on the same curve. The initial profiles are represented at  $t = t_0$ . Between  $t_0$  and  $t_1$  ( $t_1 > t_0$ ), a preferential extraction of Zr occurs involving a slight displacement of the solid/liquid interface ( $v_{s/l}$ ) and the formation of a Zr-depleted and U-rich solid oxide transition zone (T.Z). The liquid metal resulting from this first step is no longer liquid steel but corresponds



320 to a Zr-rich liquid metal. Between  $t_1$  and  $t_2$  ( $t_2 > t_1$ ), the transition zone is  
 dissolved by the surrounding liquid leading to a displacement of the solid/liquid  
 interface and the formation of a new transition zone at the interface. A U-  
 rich liquid metal near the interface is obtained by the previous dissolution of  
 the transition zone. Far from the interface, the metal liquid corresponds to an  
 325 enriched liquid in both uranium and zirconium with an atomic ratio U/Zr higher  
 than the atomic ratio of the Zr-rich liquid metal obtained at  $t_1$ . A comparison  
 can be made between this two-step process and the morphological sequence of  
 the interaction zone. In fact, the interaction time is longer in the upper part  
 of the interaction than in the lower part. On the right part of Figure 12, the  
 330 process occurring at  $t_1$  is linked with the lower part of the interaction zone  
 where only Zr-rich liquid inclusions are observed. At this position, the liquid  
 phase is discontinuous and the volume ratio  $V_{liq}/V_{sol} \gg 1$ , which implies a small  
 reactive surface and consequently a partial dissolution of the crust. Conversely  
 the upper part is linked with the second process occurring at  $t_2$  due to the small  
 335 size of the U-rich oxide grains and the composition of the liquid metal which  
 is richer in uranium. Indeed, at this position, the liquid phase is continuous  
 and the volume ratio  $V_{liq}/V_{sol} \ll 1$ , involving a large reactive surface and thus  
 causing the dissolution of the U-rich solid. Finally, the assumption of a two-step  
 dissolution process is reinforced by qualitative concentration profiles obtained  
 340 from post-test analyses and represented in Figure 13. These profiles are taken  
 at the lower part (left side) and the upper part (right side) of the interaction  
 zone. However, more concentration profiles at these different locations need to  
 be analysed in order to confirm this assumption.

From the penetration kinetics results, an effective diffusion coefficient for the  
 345 molten steel can be evaluated by :

$$D_{eff} = \frac{L_{pen}^2}{t_{exp}}. \quad (3)$$

At 1700 °C, an average value of  $4.7 \times 10^{-10} \text{ m}^2/\text{s}$  is found and appears to be  
 significantly larger than solid-state diffusion. This fast penetration could be

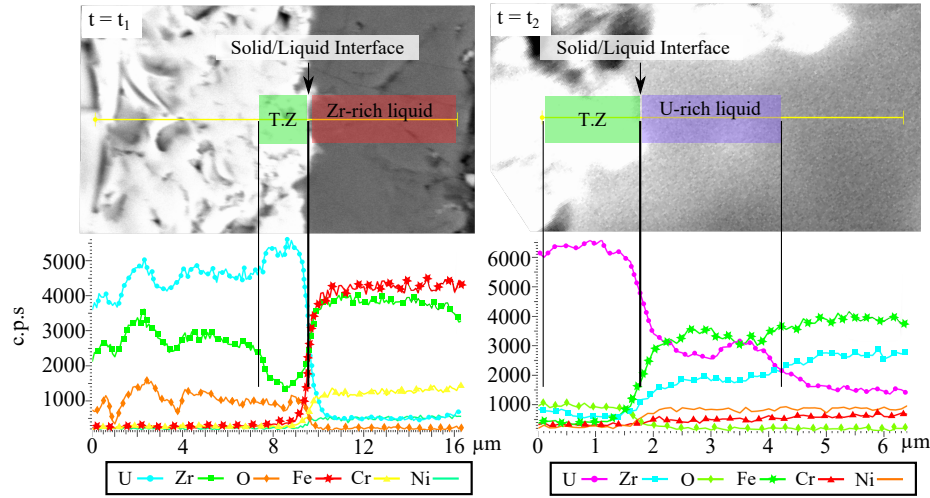


Figure 13: EDX lines of the corium crust - molten metal interface at the lower part (left panel) and the upper part (right panel) of the interaction zone (c.p.s : EDX's Counts Per Second). Corresponding BSE images are also represented.

linked to the dissolution process described in the previous paragraph. During the first step of the interaction, zirconium is preferentially leached from the crust and interacts with the molten steel. This interaction could lead to the formation of liquid metallic phase in the crust, principally at preferential paths, and to the creation of open liquid channels. These liquid channels allow molten steel to progress rapidly within the corium crust by liquid-state diffusion and to sustain zirconium leaching at the interface. Grain boundaries, sub-grain boundaries or porosities could be preferential sites to initiate the interaction (first step), after what, at a later time, the grain itself is dissolved by the surrounding liquid (second step of the interaction). The role of preferential paths could explain the heterogeneous penetration of the molten steel through the corium crust and the interconnected network of channels observed in the interaction zone. Such mechanisms have been previously observed in an experimental study dedicated to the dissolution of  $\text{UO}_2$  by molten zircaloy [17]. In addition, it is found that the penetration decelerates with increasing exposure time. The penetration kinetics could be limited by the enrichment of the molten metal in corium crust elements

(U, Zr, O). In these CORMET experiments, a tendency towards an equilibrium  
365 value of U and Zr concentration within the metallic droplet is observed. The  
deceleration of the penetration could be correlated to a chemical equilibrium  
between the corium crust and the enriched molten metal. In that case, the  
dissolution attack should stop when the asymptotic values are reached. An  
experiment with a longer exposure time should confirm this assumption and  
370 constitutes an outlook of the present work.

## 5. Conclusion and outlook

This experimental study of the interaction between suboxidized C-70 corium  
crust and molten 304L stainless steel has unveiled new results on the steel pen-  
etration through the crust and the material transport occurring during this  
375 interaction. These results show that molten steel penetrates within the crust  
according to two mechanisms observed using SEM analyses. Firstly, molten  
steel can penetrate through macroscopic defects formed during the solidifica-  
tion process, and secondly, by a dissolution mechanism. The latter is found to  
be the preponderant mechanism and forms an interconnected network of chan-  
380 nels filled by metal which delimits the interaction zone. The interaction zone  
has a particular morphological sequence with small partially dissolved corium  
grains surrounded by metal in the upper part and metal inclusions within a  
continuous corium crust in the lower part. The metal filling the channels, as  
well as the metallic droplet located above the interaction zone, are composed  
385 of steel elements (major elements : Fe, Cr, Ni) and crust elements (U, Zr, O)  
coming from the dissolution mechanism. A composition gradient in U and Zr is  
observed between the lower part of the interaction zone and the metallic droplet,  
with the latter richer in crust elements. In addition, the atomic ratio U/Zr in  
the metal shows that Zr is preferentially extracted from the crust. In order  
390 to explain these observations, a two-step process occurring at the solid/liquid  
interface is proposed. In the first step, a Zr-rich liquid metal and a U-rich solid  
transition zone are formed by the preferential extraction of Zr, which leads to a

slight motion of the interface. In the second step, the U-rich transition zone is dissolved involving an enrichment in U of the metal and a reinforced motion of the solid/liquid interface. This two-step process is correlated with a change in the volume ratio  $V_{liq}/V_{sol}$  along the morphological sequence of the interaction zone and the composition gradient. Finally, a deceleration of the penetration is observed and linked to the asymptotic behaviour of U and Zr contents in the metallic droplet with increasing exposure times.

These new insights can be used for numerical modelling of the molten pool in IVR conditions. Additional experiments and analyses are necessary before implementing these results in code dedicated to severe accidents. Experiments with longer exposure time are required in order to confirm the asymptotic behaviour of U and Zr contents in the metal and its correlation with the deceleration of the penetration. Experiments with different corium compositions are also recommended in order to highlight new mechanisms of penetration and kinetics. Future analyses will be conducted on the existing samples: EBSD analyses in order to observe the role of grain boundaries or sub-grain boundaries in the penetration process, and WDS analyses to diminish uncertainties on oxygen content. A short-term outlook of the present study would also consist in modelling the dissolution mechanism of the crust at a smaller scale than the macroscopic model recently proposed [18]. This meso-scale model will use multi-component diffusion theory as well as moving phase boundary problem and will be linked to CALPHAD-based thermodynamics calculations. This model will aim at reproducing the preferential extraction of Zr as well as the composition profiles.

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