

About nuclear fuels on MARS

P. Martin, R. Bes, E. Geiger, V. Pennisi, P. Matheron, Yves Pontillon, B. Arab-Chapelet, M. Rivenet, P.-L. Solari, S. Schlutig, et al.

▶ To cite this version:

P. Martin, R. Bes, E. Geiger, V. Pennisi, P. Matheron, et al.. About nuclear fuels on MARS. SUM2015 - Soleil User Meeting 2015, Jan 2015, Palaiseau, France. cea-02506792

HAL Id: cea-02506792 https://cea.hal.science/cea-02506792

Submitted on 12 Mar 2020 $\,$

HAL is a multi-disciplinary open access archive for the deposit and dissemination of scientific research documents, whether they are published or not. The documents may come from teaching and research institutions in France or abroad, or from public or private research centers. L'archive ouverte pluridisciplinaire **HAL**, est destinée au dépôt et à la diffusion de documents scientifiques de niveau recherche, publiés ou non, émanant des établissements d'enseignement et de recherche français ou étrangers, des laboratoires publics ou privés.

FROM RESEARCH TO INDUSTRY

ceaden

ABOUT NUCLEAR FUELS ON MARS

<u>P. Martin¹</u>, R. Bès^{1,4}, E. Geiger¹, V. Pennisi¹, P. Matheron¹, Y. Pontillon¹, B. Arab-Chapelet², M. Rivenet³, P.-L. Solari⁴, S. Schultig⁴, C. Martial¹, M. Strach¹, R. Belin¹, D. Prieur⁵

¹CEA, DEN, DEC, Centre d'études nucléaires de Cadarache, Saint Paul Lez Durance, 13108, France

² CEA, DEN, DRCP, Centre d'études nucléaires de Marcoule, Bagnols-sur-Cèze, 30207, France

³ Univ. Lille Nord de France, Unité de Catalyse et de Chimie du Solide, UCCS, UMR CNRS 8181, ENSCL-USTL, BP 90108, 59652 Villeneuve d'Ascq Cedex, France

⁴ Synchrotron SOLEIL, Ligne de lumière MARS, L'Orme des Merisiers, Saint Aubin, BP 48, 91192 Gif-sur-Yvette Cedex, France

⁵ European Commission, Joint Research Centre, ITU, P.O. Box 2340, 76125 Karlsruhe, Germany

2015 SOLEIL USER MEETING



CONTEXT

The MARS beamline is dedicated to the study of radioactive materials

- \Rightarrow in the future: Active sample (α , β , γ) up to 18.5 GBq (185 GBq per experiment)
- \Rightarrow Experimental station shielding (OK for XRD)

➡ Characterization (XRD & XAS) of irradiated fuel samples

XAS is a powerful tool to enhance the understanding of processes encountered in nuclear fuels \Leftrightarrow 3 examples of studies related to nuclear fuels:

- Development of advanced nuclear fuel : UO₂ doped with Nb *in situ* control of oxygen potential ⇔ (Vanessa Pennisi's Ph'd)
- 2. Mixed oxide samples application of emission spectrometer

→ Study of (U,Pu(Am)) O_{2+x} samples = GEN IV - Na fast reactor fuel & U-Pu-O phase diagram \Leftrightarrow (Michal Strach's Ph'd & René Bès's post-doc)

3. Fission products behavior in UO₂-SIMFUEL samples (E. Geiger's Ph'd).

Pressurized Water Reactor (2nd generation) → Uranium Dioxide fuel

22 den

In nuclear oxide fuels, the **temperature** and the **oxygen partial pressure** (i.e. the oxygen potential) are essential parameters governing the thermo-mechanical evolution of the irradiated material.



In-situ control of PO₂ thanks to the buffer capacity of an oxido-reductive dopant present under two different oxidation states.

Study of the doping and fission products thermochemistry

Ceaden Determination of the range of oxygen potential



The reactivity of fission gas depends on PO_2

Fugacity profiles and speciation of major gaseous

Objective: Fuel operating in the most favorable oxygen potential area.

\rightarrow Two selection criteria :

- Main : Minimal gaseous fraction of corrosive Fission Products (FP)
- Secondary : Highest FP immobilization

Three areas are delimited :

- An unfavorable area
- Highest risk of corrosion.

An optimum area

- Minimal fraction of gaseous corrosive FP.
- > Maximal immobilization of the FP.

An intermediate area

Limited risk of corrosion.

Ceaden Choice of the redox buffer

Criteria for the choice of the redox buffer:

- Temperature range
- Buffering capacity
- Limited solubility in the UO₂ matrix (~1000-2000 ppm),
- Final properties of the un-irradiated fuel,...

Selected dopant : NIOBIUM

Potential redox buffers	Buffering capacity (mole O / mole Nb)				
Nb ₂ O ₅ /NbO ₂	0.5				
NbO ₂ /NbO	1				
Nb ₂ O ₅ /NbO ₂ /NbO	1.5				

- Redox reactions likely to be thermodynamically activated above 1000°C
- ✓ Liquid phase Nb₂O₅ at T > 1500°C (→ grain growth)



Niobium buffering systems position compared to the stability areas of the corrosive species

MANUFACTURING CONDITIONS

- Batches with different niobium compositions (50/50 %m. for the redox couples) :
 - \blacksquare UO₂ + x wt% (NbO₂ + NbO)

<u>ceaden</u>

- UO₂ + x wt% (Nb₂O₅ + NbO₂)
- Niobium doped pellets manufacturing process :





Ceaden Results - Microstructural analysis

Ex: UO₂ + 0.8 wt% (NbO₂ + NbO) pellets

→ <u>Microstructures before annealing</u>



- Healthy pellets (no crack)
- Grain coarsening: grain size about 35 µm (10 µm classical)
- High density: 94.2%Td

Elemental EDX maps







 Micrometer size Nb oxides precipitates at grain boundaries and porosities

Ceaden Results - Microstructural analysis

- Objective: Elemental composition of Nb-O precipitates
- → SEM + EDX analyzes
- Sample annealed at 1000°C



- Two different color contrasts
- Change of the niobium content from one grey to another (composition line)
- → Presence of two different niobium/oxide ratio domains
- \rightarrow 2 phases or more complex system ?

➔ µ-XAS experiment



Profile and element contents

Same conclusion for annealing at **1200°C** (two different niobium oxide phases)

Ceaden µ-XRF AND µ-XANES RESULTS

Experiment performed on MARS - 4 days in mid-November

- Nb K edge (18986 eV)
- Micro-focussing using 2 mirrors in K&B geometry → 10⁹ ph.s⁻¹ at 19000 eV
- μ -XRF elemental mapping & μ -XAS using a beam 12x13 μ m²

⇔ U Lα (13614 eV) & Nb K (16615 eV)

<u>Aims</u>:

- Nb chemical state in precipitates \rightarrow μ -XANES = buffer effectiveness
- Nb speciation inside UO₂ matrix \rightarrow µ-XANES + µ-EXAFS



Reference	E ₀ (eV)				
Nb(0)	18986.0				
NbO - Nb(+2)	18994.1				
NbO ₂ - Nb (+4)	19001.7				
Nb ₂ O ₅ - Nb (+5)	19004.0				

Ceaden µ-XRF AND µ-XANES RESULTS ON PRECIPITATES

Sample S1: UO₂ + 0.8 wt% (NbO₂ + NbO) as fabricated

2 precipitates studied



Ceaden µ-XRF AND µ-XANES RESULTS ON PRECIPITATES

Sample S1: $UO_2 + 0.8$ wt% (NbO₂ + NbO)



Annealed 1 hour at 1000°C



- → E₀ systematically between NbO and NbO₂ positions
- → Shoulder at low energy = metallic Nb ?

→ μ -XANES spectra are reproduced by linear combination of Nb + NbO and NbO₂

Ceaden µ-xrf and µ-xanes results on precipitates



➔ The two phases NbO₂/NbO are observed <=> buffering capacity confirmed

➔ Unexpected metallic Nb signal systematically observed !

Ceaden µ-xrf and µ-xanes results on matrix

 $\mu\text{-XANES}$ & $\mu\text{-EXAFS}$ on the matrix :

• same spectra observed on every sample (S1 and S2, annealed or not)



- → Different with known Nb oxides ⇔ ≠symmetry
 - ➔ Consistent with Nb inside UO₂ fluorite structure ?
 - → E_0 position ~ between Nb⁺⁴ / Nb⁺⁵

 μ -EXAFS results => signal up to 11.5 Å⁻¹



- Signal from hidden precipitates can not be excluded
- Longer Nb-O distance (Nb-O ≤ 2.15 Å in Nb oxides)
- \Rightarrow U-O distance in UO₂ is 2.37 Å.

Use of an emission spectrometer with radioactive materials ?

- Remove Bragg peaks and isolate emission lines for diluted elements in fuels: dopant (Cr, Gd), fission products (Xe, Kr, Cs, Ba, ...)
- Radioactive background, emission lines due to other elements



Up to 4 crystals (Si, Ge)



L₃ edge (17,166 eV) core-hole = 8.13 eV L α 1(3d_{5/2}→2p_{3/2}) => resolution = 3.55 eV First experiment on Mixed oxides samples in july 2014

- UO₂, U₃O₈, PuO₂ containing 1.7 % Am (due to ²⁴¹Pu decay)
- Dense sintered (U_{0.45}Pu_{0.542}(Am_{0.008}))O₂ pellets prepared with conventional powder metallurgy process





[1] C. Guéneau et al., J. Nucl. Mater. 419 (2011) 145-167

[2] Sari et al. , J. Nucl. Mater. 35 (1970) 267-77

The hyper-stoichiometric domain at RT



 M_4O_9 + hexagonal M_3O_8 biphasic domain – Boundaries ? Pu content in M_3O_8 controversial M_3O_8 + MO_{2+x} biphasic domain ?

Study of U-Pu-O phase diagram

- HT-XRD experiments performed at the LEFCA facility (CEA Cadarache)
- Hyper-stoichiometric domain : up to 1500°C under air
- $\Rightarrow (U_{0.45}Pu_{0.542}(Am_{0.008}))O_{2+x}$ domain = MO_{2+x} phase at RT
- \Rightarrow U, Pu and Am L₃ HERFD XANES



[1] K.O. Kvashnina et al. Phys. Rev. Lett. 111 (2013) 253002

Sample	E ₀ (eV)				
UO ₂ – U ⁴⁺	17170.3				
U ₃ O ₈ – U ⁵⁺ +U ⁶⁺ [1]	17172.2				
(U _{0.45} Pu _{0.55})O ₂	17170.2				
(U _{0.45} Pu _{0.55})O _{2+x}	17171.5				

- $(U_{0.45}Pu_{0.55})O_2 => UO_2 = U^{+4}$
- (U_{0.45}Pu_{0.55})O_{2+x}
 => between UO₂ & U₃O₈
 => Mixed U valence = +4/+5/+6

Study of U-Pu-O phase diagram

- HT-XRD experiments performed at the LEFCA facility (CEA Cadarache)
- Hyper-stoichiometric domain : up to 1500 °C under air
- $\Rightarrow (U_{0.45}Pu_{0.542}(Am_{0.008}))O_{2+x}$ domain = MO_{2+x} phase at RT
- \Rightarrow U, Pu and Am L₃ HERFD XANES



- No modification = PuO₂ position
- As expected Pu⁺⁴
- Cubic fluorite symmetry around Pu is not modified compared to U

Study of U-Pu-O phase diagram

- HT-XRD experiments performed at the LEFCA facility (CEA Cadarache)
- Hyper-stoichiometric domain : up to 1500 °C under air
- $\Rightarrow (U_{0.45}Pu_{0.542}(Am_{0.008}))O_{2+x}$ domain = MO_{2+x} phase at RT
- \Rightarrow U, Pu and Am L₃ HERFD XANES



Sample	E ₀ (eV)				
PuO ₂	18515.7				
(U _{0.45} Pu _{0.55})O ₂	18512.1				
(U _{0.45} Pu _{0.55})O _{2+x}	18512.2				

- Ref = Am in PuO_2 (²⁴¹Pu decay)
- Reduction to Am⁺³
- Am not modified by oxidation process !!

Am³⁺ in hyper-stoichiometric MOX

Study of U-Pu-O phase diagram

- HT-XRD experiments performed at the LEFCA facility (CEA Cadarache)
- Hyper-stoichiometric domain : up to 1500 °C under air
- \Rightarrow (U_{0.45}Pu_{0.542}(Am_{0.008}))O_{2+x} domain = MO_{2+x} phase at RT
- \Rightarrow U, Pu and Am L₃ HERFD

Conclusion

- Actinides emission line of each actinide was successfully isolated
- The stoichiometric MOX = fluorine structure $MO_2 = U^{+4} Pu^{+4} + Am^{+3}$
- The hyper-stoichiometric is more complex than expected compared to XRD
 => mixed valence U⁺⁴ U⁺⁵ U⁺⁶ / Pu ⁺⁴ / Am⁺³ ⇔ MO_{2+x} domain
- Study of different Pu content => M_3O_8 domain

Ceaden Fission Products Behavior

During a severe accident, irradiated nuclear fuels are submitted to High Temperature (>3000K) + Oxidizing/reducing atmosphere ex: Fukushima Daiichi (Japan 2011)

The determination of the term source (amount of radionuclide released from the core) is directly connected to the fission products (FP) behavior

• FP chemical state evolution during the accident and identification of intermediary phases

 \rightarrow UO₂ sample doped with stable non volatile FP during manufacturing

➔ Preparation of future experiment on spent fuels

Sample composition

Burnup	Composition (At. %)										
(GWd/t)	Ва	Ce	La	Мо	Sr	Y	Zr	Rh	Pd	Ru	Nd
76	0.26	0.61	0.20	0.51	0.13	0.06	0.60	0.03	0.42	0.64	0.91

Sample annealed 1 hour at 1700°C under Ar/H₂

Ceaden Fission products behavior

- First μ -XANES experiment on MARS = 11/2014
- Focused on Mo behavior
- \Rightarrow Mo K edge (20000 keV)
- \Rightarrow µ-XRF & µ-XANES \Leftrightarrow 6x7 µm² with 5.10⁸ ph.s⁻¹ at 20 keV

SEM picture



SEM/EDS Mo L_{α} map



UO₂ spent fuel



Geaden Fission PRODUCTS BEHAVIOR

First μ -XANES experiment on MARS = 11/2014

Focused on Mo behavior => μ -XRF & μ -XAS (6x7 μ m² 5x10⁸ Ph/s at 20 keV)



Mo is mostly metallic but difference observed most probably due to association with other fission products (Ru, ...)

=> More details during Ernesto Geiger presentation (Chemistry session at 10h30)

FROM RESEARCH TO INDUSTRY



Thank for your attention

FISSION PRODUCT CHEMISTRY ON NUCLEAR FUEL

Ruthenium K-edge

Form clusters (with Mo)

cea den

1 transition collected (K-L₃)
 Ru Kα₁ HERDF-XANES





- No evolution of Ru local environment
- Metallic Ru
- Probable mixing of Mo/Ru metals
- Ab initio calculations under progress

Petit-déjeuner scientifique, SOLEIL synchrotron- 12/05/2014

Use of an emission spectrometer with radioactive materials ?

- Remove Bragg peaks and isolate emission lines for diluted elements in fuels: dopant (Cr, Gd), fission products (Xe, Kr, Cs, Ba, ...)
- Radioactive background, emission lines due to other elements



Up to 4 crystals (Si, Ge)





L₃ edge (17,166 eV) core-hole = 8.13 eV La1($3d_{5/2}$ →2p_{3/2}) => resolution = 3.55 eV