

Effect of chromium grain size and morphology on the HT oxidation behavior of chromium coated Zr based alloys

Nihed Chaabane-Jebali, Jean-Chistophe Brachet, Matthieu Le Saux, Didier Hamon, Elodie Rouesne, Stéphane Urvoy, Thomas Guilbert, Michel Schlegel, Michel Tabarant, Fernando Lomello

▶ To cite this version:

Nihed Chaabane-Jebali, Jean-Chistophe Brachet, Matthieu Le Saux, Didier Hamon, Elodie Rouesne, et al.. Effect of chromium grain size and morphology on the HT oxidation behavior of chromium coated Zr based alloys. Materials Science and Engineering - session Structural Materials and Metallurgy, Feb 2019, San Francisco, United States. cea-02394049

HAL Id: cea-02394049 https://cea.hal.science/cea-02394049

Submitted on 21 Feb 2020 $\,$

HAL is a multi-disciplinary open access archive for the deposit and dissemination of scientific research documents, whether they are published or not. The documents may come from teaching and research institutions in France or abroad, or from public or private research centers. L'archive ouverte pluridisciplinaire **HAL**, est destinée au dépôt et à la diffusion de documents scientifiques de niveau recherche, publiés ou non, émanant des établissements d'enseignement et de recherche français ou étrangers, des laboratoires publics ou privés.



EFFECT OF CHROMIUM GRAIN SIZE AND MORPHOLOGY ON HIGH TEMPERATURE STEAM OXIDATION BEHAVIOR OF CHROMIUM COATED Zr BASED ALLOYS



Nihed Chaâbane^a, Jean-Christophe Brachet^b, Matthieu Le Saux^b, Didier Hamon^b, Elodie Rouesne^b, Stéphane Urvoy^b, Thomas Guilbert^b

Michel Schlegel^c, Michel Tabarant^{c,} Fernando Lomello^c

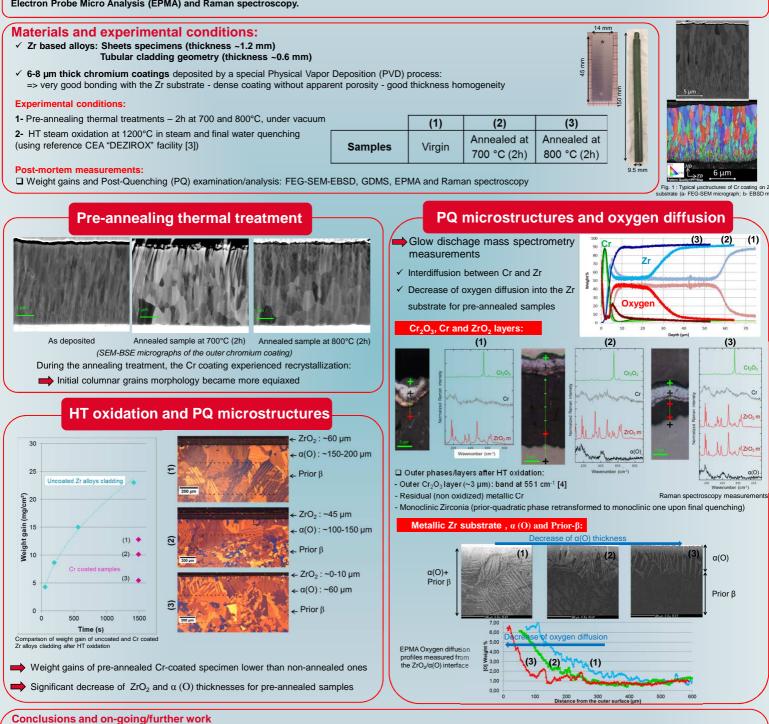
^a INSTN, CEA, Université Paris-Saclay, F-91191, Gif-sur-Yvette, France

^b DEN, CEA, Université Paris-Saclay, Service de Rech. Métall. Appliquées, F-91191, Gif-sur-Yvette, France

^b DEN, CEA, Université Paris-Saclay, Service d'Etudes Analytiques et de Réactivité des Surfaces , F-91191, Gif-sur-Yvette, France

Context: Chromium coated zirconium based nuclear fuel claddings are developed and studied within the CEA-Framatome-EDF joint research program as "Enhanced Accident Tolerant Fuel" (EATF) cladding short-term concept for nuclear light water reactors [1] [2].

Objective: To investigate the influence of the initial chromium grain size and morphology on the High Temperature (HT) steam oxidation behavior of 6-8µm thick Crcoated zirconium based specimens, by coupling several *post-mortem* experimental techniques, that is, FEG-SEM-EBSD, Glow discharge Mass Spectrometry (GDMS), Electron Probe Micro Analysis (EPMA) and Raman spectroscopy.



- Thermal pre-annealing treatment results in an increase of Cr size grains before the HT oxidation
- > Weight gains of pre-annealed Cr-coated specimen were lower than the non-annealed ones.
- Thickness of ZrO₂ and αZr(O) phases were significantly reduced for the pre-annealed samples, indicating that the grain size and morphological evolutions of the chromium coating upon the pre-annealing thermal treatment may influence its further HT oxidation behavior, thus inducing:
 - Overall, higher resistance to HT oxydation
- Significant delay in the oxygen ingress into the Zr based substrate
- On-going: complementary EBSD, TEM... analysis for more detailed informations on the associated oxygen diffusion mechanisms...

Acknowledgments: Framatome and EDF for financial support and fruitful discussions

Some references:

- 1. J.C. Brachet, et al., "Early studies on Cr-Coated Zircaloy-4 as Enhanced Accident Tolerant Nuclear Fuel Claddings for Light Water Reactors", submitted to Journal of Nuclear Mat. (2019)
- 2. J. Bischoff, et al., "AREVA NP's Enhanced Accident Tolerant Fuel Developments: Focus on Cr-coated M5 Cladding", Nuclear Engineering and Technology 50, (2018), 223-2283
- 3. J.C. Brachet et al., "Mechanical behavior at Room Temperature and Metallurgical study of Low-Tin Zy-4 and M5TM alloys after oxidation at 1100°C and quenching", IAEA-TECDOC-1320 (2001)
- 4. I. Idarraga, et al., "Raman investigation of pre- and post-breakaway oxide scales formed on Zircaloy-4 and M5R in air at high temperature", Journal of Nuclear Mat. (2012) 421 160-171