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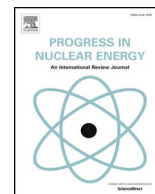
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Prospects in China for nuclear development up to 2050

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ABSTRACT

An ambitious plan for nuclear development exists for long in China. The study of scenarios with prospect of 150 GWe to 400 GWe in 2050 is carried out using the COSI6 simulation software, and aims at analyzing the evolution of nuclear energy currently planned in China. Results rely on natural uranium supplies, fuel fabrication, spent fuel reprocessing, quantities of proliferating materials and the opportunity of a rapid deployment of fast reactors (FBR). It seems impossible for China to start two fast reactors before 2020 without any external source of plutonium. Anyway, FBR may represent at the most around 30% of the total nuclear capacity in the country by 2050. Indeed the deployment of FBR only can start from 2035 to 2040. Finally, the pace of FBR development should be controlled carefully by the proportion of FBR and PWR with respect to the reprocessing capacity. Natural uranium savings appear rather low by 2050, because the transition toward a fast reactor fleet independent from uranium ore lasts decades. However, this independence may be reached by the end of the century, before uranium resources are dwindling, if the first corners to close the fuel cycle are turned in the next decade.

1. Introduction

With the rapid development of the Chinese economy, the energy demand has been increasing year by year from the beginning of the 21st century. The average electricity consumption in China is about 4000 kWh per person per year at present, but may reach 7000 kWh by the end of the year 2050, assuming an increase ratio of 1.6% annually (CNPC Economics and Technology Research Institute, 2015). Fossil fuels (especially coal) have been used as the main energy resource to meet the energy demand in China up to now. This has detrimental effects on environment and people life conditions, such as global warming and air pollution. In order to decrease CO₂ emissions in the future and alleviate the pressure on the environment caused by fossil fuel consumption, new energies have to be economically feasible, environmental friendly, publicly acceptable, and implementable at a large scale. In this respect, nuclear energy constitutes a top-ranking option for China (Zhou, 2010; Yi-chong, 2008; Zhou and Zhang, 2010; Zhou et al., 2011), along with renewable sources of energy. Indeed, to solve air pollution issues, nuclear power can efficiently balance the intermittency of renewables without increasing GHG emissions.

In 1996, the China National Nuclear Company (CNNC) forecasted that China would reach 150 GWe by 2050 (Suttmeier and Evan, 1996; Rothwell, 2001). The Medium and Long-term Nuclear Power Development Plan (2005–2020) planned in 2006 that China's nuclear capacity

would rise to about 40 GWe by 2020 (NDRC). Three years later, the status of Chinese nuclear energy development shifted from “active” to “aggressive” (Guobao, 2009). In 2011, the Chinese Academy of Engineering (CAE) predicted that nuclear power capacity would be 200 GWe in 2030 and 400 GWe in 2050 (CAE-Chinese Academy of Engineering,). But, because of the Fukushima accident, the State Council of China suspended new civilian nuclear projects and ordered safety inspections of all nuclear power plants in operation and under construction.

Over one year later, China restarted nuclear plants and approved a new Medium and Long term Nuclear Power Development Plan (2011–2020) which outlined its goal of increasing the nuclear capacity to about 58 GWe by 2020, with 30 GWe of reactors under construction to this term (National Development and Reform Commission). There is currently no official target for nuclear energy after 2020, but a few growth prospects have been proposed in the literature (Yi-chong, 2008; Mi, 2008; Energy Research Institute, 2010; Zhou, 2011; Chen et al., 2012; Fiori and Zhou, 2015a, 2015b), in which the installed nuclear capacities are between 150 GWe and 500 GWe in 2050. By 2030 and by 2050, nuclear power is expected to contribute respectively to 15% and 22% of the Chinese electricity production (CNPC Economics and Technology Research Institute, 2015).

The issue for China is not whether the country will expand its nuclear energy programme; rather it is about the speed at which this

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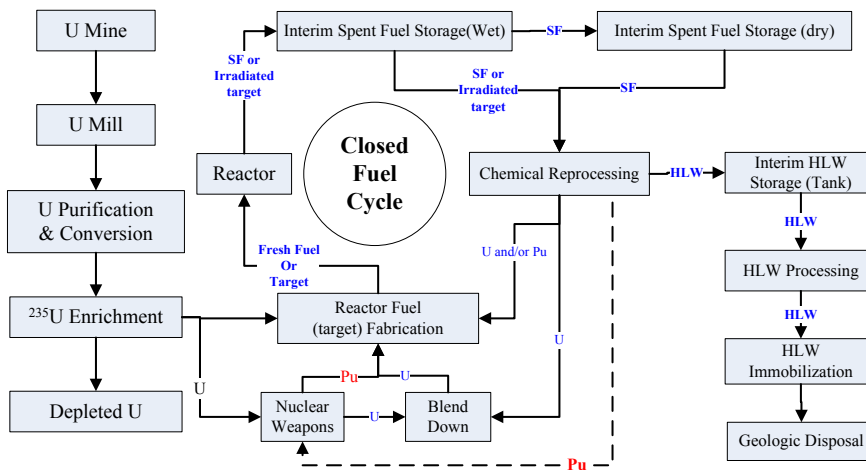


Fig. 1. Closed fuel cycle policy in China.

expansion should occur. If China would have decided to operate its nuclear fleet according to a “one-through” cycle in the future, how much natural uranium and interim storage capacity of unloaded spent fuel (SF) would be needed? If the nuclear fuel cycle is being closed, how make a reasonable use of Pu and limit the proliferation risks? What may be the short-term alternative strategies if FBR are further delayed? In order to solve the above challenges so as to ensure a sustainable development of nuclear energy in China, it is necessary to have a complete overview of the nuclear energy development in this country. To this aim, this study has been carried out with the COSI6 simulation software (Coquelet-pascal et al., 2015), to compare several scenarios built with great care as regards the level of details.

2. Nuclear industry in China

2.1. Current situation

China has aimed at closing its nuclear fuel cycle (see Fig. 1) as soon as it has decided to develop its nuclear power, in the early 1980s (Zhou, 2011). Pressurized water reactors (PWR) have been first deployed, and China is facing a considerable spent fuel accumulation. Qingshan Phase I and Dayabay sites have recently accumulated SF in excess to their storage capacity. Dense-pack racks are now used, but at least 104 irradiated assemblies were sent from Dayabay to an interim storage pool at the pilot-scale reprocessing plant (Zhou, 2011), which capacity is limited to 550 tHM (Guoqing, 2010). Therefore, the SF interim storage is becoming a critical issue for China.

A commercial reprocessing plant should be built soon to recycle spent UOX: extracted U & Pu would be used to fabricate MOX fuel for fast reactors (FBR) and/or PWR (Fiori and Zhou, 2015b). With enough SF and reprocessing capacity, the nuclear power plants (NPP) may gradually switch to FBR only, and accelerator-driven systems might be used to burn nuclear wastes (Fiori and Zhou, 2015a).

By the end of 2014, China operated different types of PWR and sets about developing FBR. 23 NPPs with total installed capacity of 19 GWe were in operation, including an experimental FBR of 25 MWe named CEFBR. Other 28 PWRs with total installed electrical capacity of 30 GWe were under construction. More information about Chinese NPPs can be found in the IAEA website (IAEA). The construction period of PWR is 5–6 years in China. 10 PWRs built in 2010 demonstrated that the capability of NPP construction, equipment manufacturing and technician training notably meets the challenge of a rapid nuclear energy development. Imported reactors are M310 3-loop PWR and EPR from France (Martin and Girieud, 2016), VVER AES 91 from Russia, CANDU 6 from Canada and AP1000 from USA. At present, China has developed its own types of PWR: CNP-300, CNP-600, CNP-1000, CPR-1000, CAP1400 and Hualong I, now mature for export (China's nuclear

export drive, 2017). A prototype of high temperature gas-cooled reactor (HTGR) of 210 MWe designed in-house by INET (Tsinghua University) should reach criticality before 2020.

For the front-end of nuclear fuel cycle, according to IAEA (IAEA, 2014), uranium resources in China totalize 265,500 tHM. Besides that, China National Nuclear Overseas Uranium Resources Development company (NNOURD) was set up to seek out and invest in overseas resources in 2006. It has secured over 200,000 tHM reserves in total in Africa, Australia, Canada, and Central Asia (Zhou, 2011). China General Nuclear Power Corporation (CGNPC) also invested in a uranium mining project in Kazakhstan in 2009. In addition, AREVA and CAMECO signed nuclear fuel agreements with CGNPC to supply low-enriched uranium fuel for the next decades (IAEA, 2014; WNN, 2010). Identified natural uranium resources recoverable at costs lower than 130 USD/kgU (50 USD/lb of U_3O_8) are 5,902,900 tU in 2013 according to IAEA (IAEA, 2014).

Three uranium conversion plants each with annual capacity of 500–800 tU have been in operation from 2002 (Energy Research Institute, 2010; WNN, 2010). Another conversion plant with annual capacity of 3000 tU is in operation from 2008 (CAEA, 2008). According to (Zhou, 2011), there are two enrichment plants with more than 1.5 million separative work units (SWU) per year in China. Two nuclear fuel manufacturers (in Yibin and Baotou) currently provide nuclear fuel assemblies (over 1,200tHM/year). A pilot-scale MOX fabrication plant with an annual capacity of 0.5 ton has just been completed.

For the back-end of nuclear fuel cycle, the State Council approved in 1986 the construction of a pilot-scale test reprocessing plant with annual capacity of 50 tHM in Gansu province (Zhou, 2011). A hot test has been successfully performed in December 2010 (CNNC, 2010), but the plant is currently under improvement for 3 years (Zhongmao, 2014). The advanced two-cycle PUREX process is employed to recover U and Pu, and trans-uranics in the PUREX raffinate are partitioned by means of TRPO process (Zhu et al., 1992). The partitioned mixture of Am, Cm and lanthanides (Ln) may be further separated by the Cyanex 301-based process if the separated trans-plutonium elements are stored for transmutation (Chen et al., 1996). The raffinate, HLLW (high level liquid wastes) and fission product lanthanides will be immobilized in glass packages using a process similar to the one used in France (Tiphine et al., 2015).

2.2. Prospects for nuclear development after 2020

Reprocessing plants and FBR are the key facilities for China to close the nuclear fuel cycle. In this objective, the design of the demonstration FBR CFR-600 (Yang et al., 2015), which should be built in XiaPu, Fujian province, has recently been completed. Two Russian BN-800 reactors (FBR of 800 MWe) are planned for construction at Sanmin and should

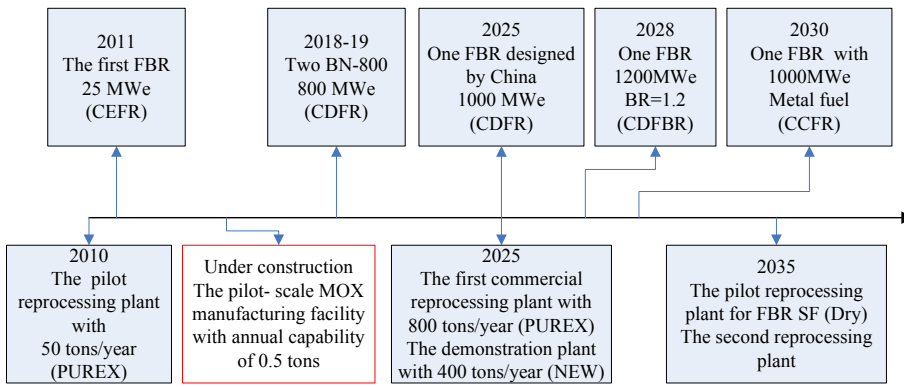


Fig. 2. Planned reprocessing plants and FBR development in China.

have initially been commissioned in 2018-19, but now a delay of few years is expected (Fiori and Zhou, 2015a). After that, the CDFBR of 1000 MWe is expected to reach criticality in 2025. Then the CDFBR of 1200 MWe, with a breeding ratio of 1.2, is projected for 2028, followed in 2030 by the first commercial fast reactor of 1000 MWe called CCFBR, which might be supplied with metal fuel (National Development and Reform Commission; Chen et al., 2012; Fiori and Zhou, 2015a).

According to Chen et al. (2012), the first large commercial reprocessing plant based on the current PUREX process will be put into operation around 2025. At the same time, a demonstration plant based on advanced reprocessing technologies will be commissioned, with a capacity of 400 tHM/yr at the most. The second large commercial reprocessing plant and a pilot reprocessing plant for FBR SF should start around 2035. The planned schedule is summarized in Fig. 2.

3. Simulation

3.1. CEA nuclear fuel cycle simulation platform

COSI is a scenario code developed by CEA, Cadarache, France, since 1985 (Coquelet-pascal et al., 2015). It simulates a fleet of NPPs and the associated fuel cycle facilities. COSI was designed to study short, medium and long-term options for the introduction of various types of nuclear reactors. In the frame of the French Act for waste management, scenario studies are carried out with COSI.

The flow scheme of COSI is illustrated in Fig. 3. COSI physical models are detailed in (Coquelet and Kieffer, 2011): the evolution calculation is performed by a coupling with CESAR (Simplified Evolution Code Applied to Reprocessing) (Vidal et al., 2012), which is the reference code used at the AREVA La Hague reprocessing plant. CESAR calculates notably the isotopic composition of irradiated fuels from their initial isotopic composition, fuel burn up and irradiation time. The neutronic data libraries (cross sections) employed by COSI to fulfill the above calculation are calculated by the CEA reference codes for neutron physics: APOLLO2 (Santamarina et al., 2009) for thermal spectrum systems and ERANOS (Rimpault et al., 2002) for fast spectrum systems.

3.2. Reactor data

The neutronic data libraries employed by COSI are here derived from French PWR and FBR core concepts. French PWR should well approximate Chinese reactors because most of PWR under operation and construction are designed on the basis of M310, imported from France. Chinese commercial FBR is currently still being designed. The reactor characteristics initially used for the simulations are summarized in Table 1. Most of them are quoted from the IAEA website (IAEA). A representative PWR named GEN-III shown in Table 1 will be used to simulate any PWR started after 2020. PWR and FBR reactors operating or planned in the future that are used as input parameters in the simulation model are all under or going to be under IAEA safeguards.

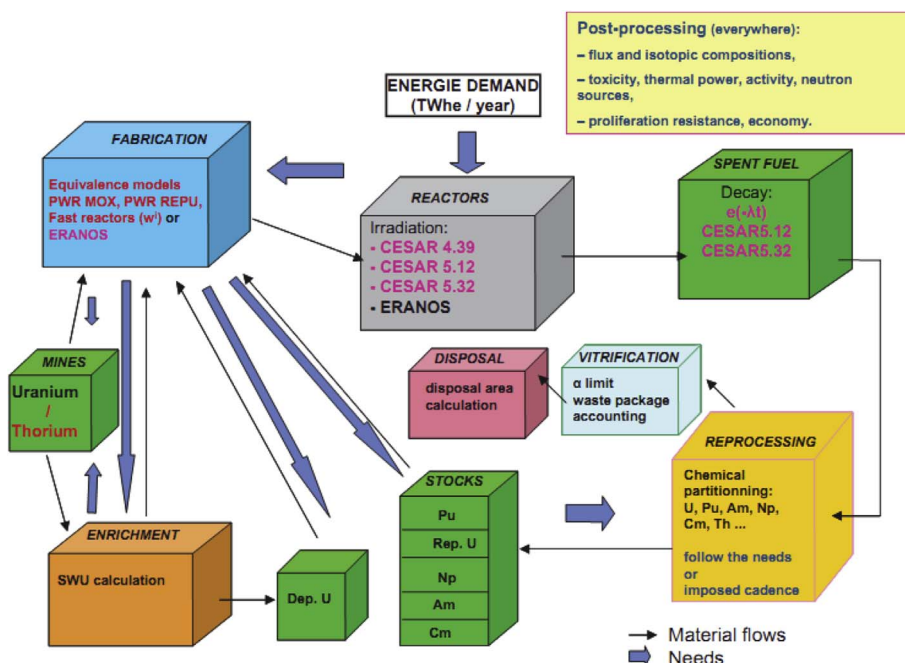


Fig. 3. COSI6 flow scheme.

Table 1
Simulation parameters of reactors in China (in bold are defined the mean characteristics of PWR after 2020).

Reactor Type	Thermal power (MWe)	Electric power (MWe)	Net Yield (%)	EFPD (days)	Cycles	Enrichment (%)	Burn -up (MWd/t)	Fuel mass per cycle (t)	Total fuel mass (t)	Load factor (%)
CNP-300	1035	330	31.9	410	3	3.4	34,000	12.5	37.5	IAEA data
CNP-650	1930	650	33.7	340	4	3.7	45,000	14.6	58.4	IAEA data
CPR-1000	2905	1000 ^a	34.7	460	4	4.45	52,000	25.7	102.8	IAEA data
AES91(VVER)	3000	990	33	325	4	4.45	52,800	18.5	74	IAEA data
M310	2905	944	32.5	460	4	4.45	52,000	25.7	102.8	IAEA data
AP1000	3400	1250	37.0	518	3	4.45	55,000	32	96	87
EPR	4300	1500	34.9	518	3	4.2	51,800	43	129	87
GEN-III (Assumption)	3400	1100	34.5	518	3	4.45	55,000	30	90	87
SFBR -600 (fissile core)	1407	562.8	40	533	5	–	100,000	7.5	37.5	87
SFBR -600 (axial fertile blanket)	51	20.4	40	533	5	–	10,000	2.7	13.5	87
SFBR -600 (radial fertile blanket)	43	17.2	40	533	10	–	10,000	2.3	23	87
SFBR -1500 (fissile core)	3524	1410	40	450	5	–	100,000	14.8	74	87
SFBR -1500 (axial fertile blanket)	129	51.6	40	450	5	–	10,000	5.4	27	87
SFBR -1500 (radial fertile blanket)	119	47.6	40	450	10	–	10,000	5	50	87

^a The value used to simulate the scenarios is 944 (see section 4.1).

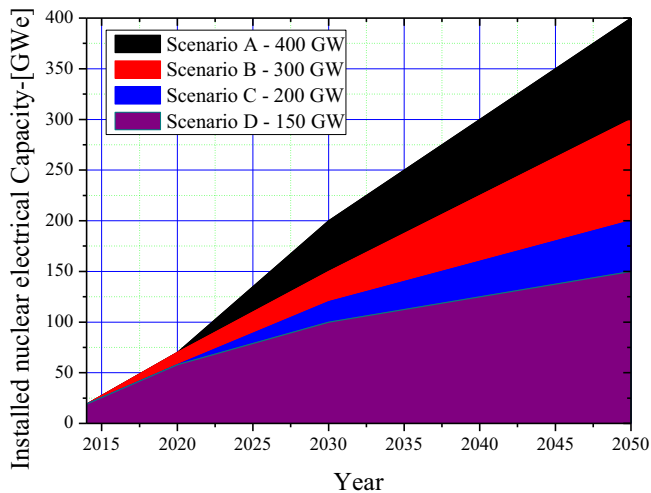


Fig. 4. Various evolutions of the nuclear power up to 2050.

3.3. Scenarios

Four scenarios in terms of electricity production have been studied. Two values of installed nuclear electrical capacity for 2020 have been used: 58 GWe (National Development and Reform Commission) and 70 GWe (CAE-Chinese Academy of Engineering). The scenario D comes to 150 GWe in 2050 (Suttmeier and Evan, 1996; Rothwell, 2001) whereas scenario A reaches 400 GWe (CAE-Chinese Academy of Engineering; Chen et al., 2012; Fiori and Zhou, 2015a, 2015b). The scenarios B and C (Mi, 2008; World Nuclear Association, 2015; Evans et al., 2015) stand between, with 300 GWe and 200 GWe respectively. Assuming a linear growth from 2020 to 2050, the curves of nuclear electrical capacity can be drawn in Fig. 4. Since the total electricity demands by 2050 should be 12,000–15,000 TWh according to the data published by NEA (Chinese

National Energy Administration), the electrical production for scenarios A, B, C and D accounts for 21–26%, 15–19%, 10–13% and 8–10% of the total electricity demand, respectively.

Three different options, namely “one-through” way (OTC), “Pu recycled in PWR” and “Pu recycled in FBR”, which are illustrated in Figs. 5–7, have been simulated. The evolution of the Chinese nuclear fleet during the 4 scenarios A, B, C and D is presented in Table 2. There are 52 PWRs which should be in operation or under construction before 2020: detail information can be found in the IAEA website (IAEA). All newly built PWRs are CPR-1000 reactors in scenarios A and B before 2020, and GEN-III reactors after. All FBRs are SFBR-1500 reactors, except 2 SFBR-600 built in 2025.

In the case of Pu recycled in PWR, there are always 13 PWRs and 2 SFBR-600 loaded with MOX fuel, with one reprocessing plant with a capacity of 800 tHM/year commissioned in 2025. The number of PWRs loaded with MOX fuel is determined by the annual Pu output from the reprocessing plant. This output is approximately 8 tons. According to the calculated results, 13 PWRs cannot be loaded with MOX fuel in 2026, so that some of them are loaded with MOX fuel in 2027.

In short, here are the applied assumptions used to simulate the scenarios, which start from the beginnings of the nuclear power in China (1991):

1. After 2020, only GEN-III PWR and FBR are commissioned.
2. The FBR and reprocessing plants are developed as planned in Fig. 3 when applying the “Pu recycled in FBR” strategy.
3. SFBR-600 substitute for BN-800 and SFBR-1500 for CDFBR, CDFBR and CCFBR shown in Fig. 2. All FBR have a breeding ratio of around 1.2.
4. The cooling time in onsite pool is 5 years for UOX SF and 3 years for MOX SF (i.e. 1 year more than in (Gao et al., 2015)). Because MOX SF cool down only for 3 years before reprocessing, the radiolysis of the TBP extractant is likely to occur due to high radioactivity. Therefore, some new technology like high-temperature electrical

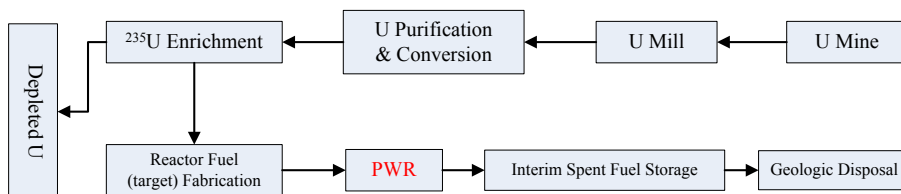


Fig. 5. “One-through” way.

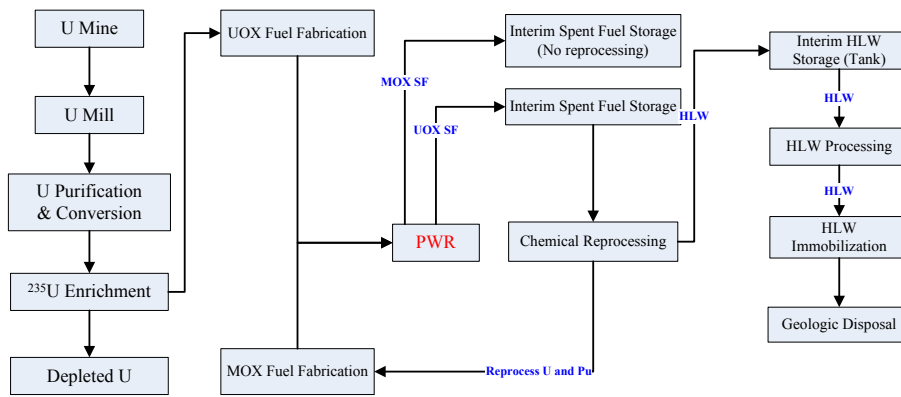


Fig. 6. “Pu recycled in PWR”.

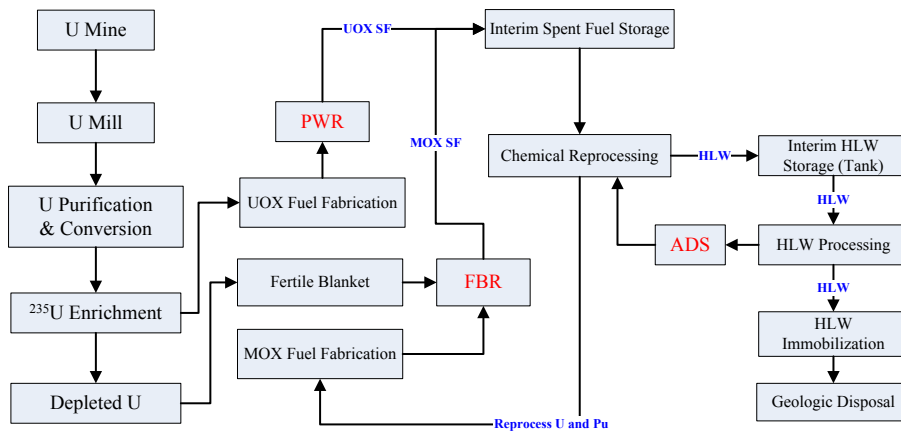


Fig. 7. “Pu recycled in FBR”.

Table 2
Distribution of PWR and FBR for all the simulations in 2050.

Scenario	OTC	Pu recycled in PWR	Pu recycled in FBR
A	52 PWR + 313 GEN-III	352 PWR + 13 PWR loaded with MOX + 2 SFBR-600	246 PWR + 94 FBR
B	52 PWR + 222 GEN-III	261 PWR + 13 PWR loaded with MOX + 2 SFBR-600	194 PWR + 64 FBR
C	52 PWR + 140 GEN-III	179 PWR + 13 PWR loaded with MOX + 2 SFBR-600	133 PWR + 45 FBR
D	52 PWR + 95 GEN-III	134 PWR + 13 PWR loaded with MOX + 2 SFBR-600	100 PWR + 35 FBR

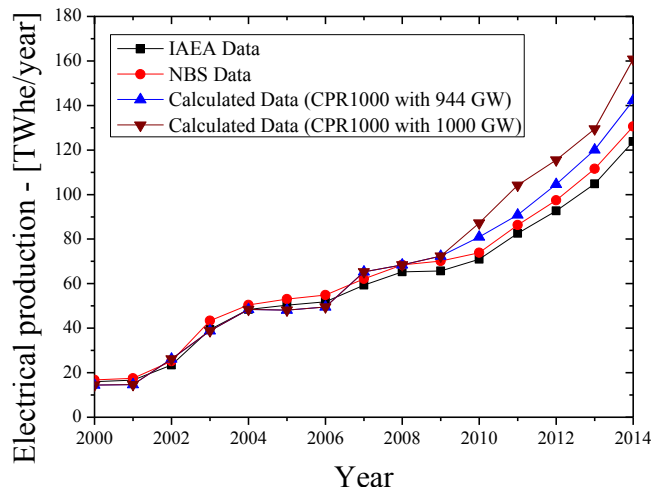


Fig. 8. Electricity production up to 2014.

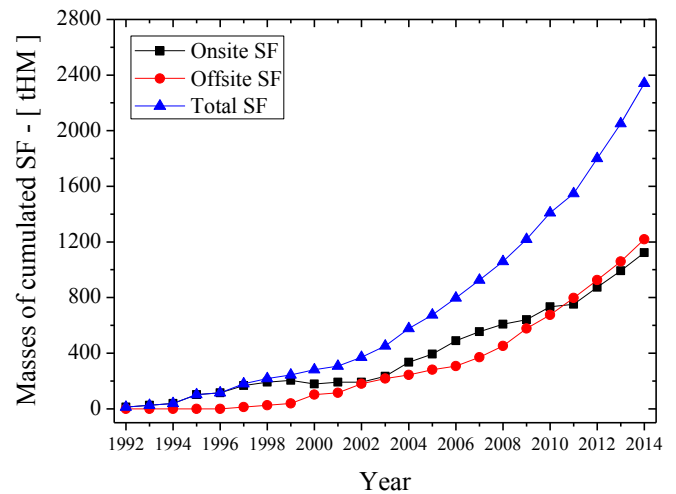


Fig. 9. Cumulated SF calculated by COSI up to 2014 (without CANDU SF).

chemistry reprocessing might be put into use if this cooling period applies.

- Material losses in conversion, enrichment, fabrication, reprocessing, MA separation and transmutation are neglected.

4. Results and discussion

4.1. Calculation assessment

The simulated electricity production between 2000 and 2014 has been compared with the data from the National Bureau of Statistics (NBS) of the People's Republic of China and from IAEA. The results are

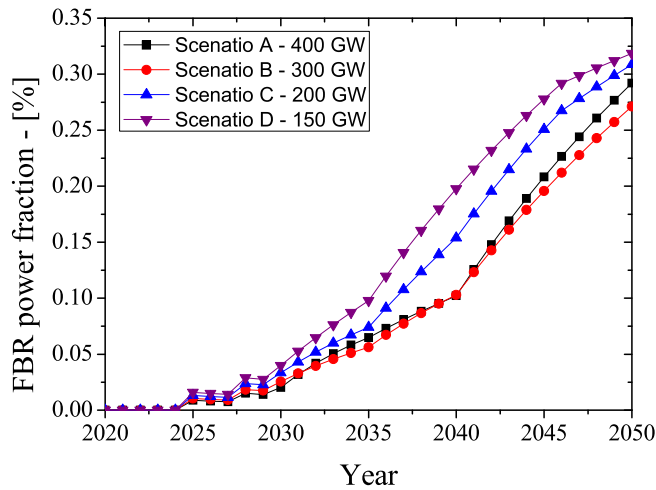


Fig. 10. FBR power fraction for “Pu recycled in FBR” scenarios.

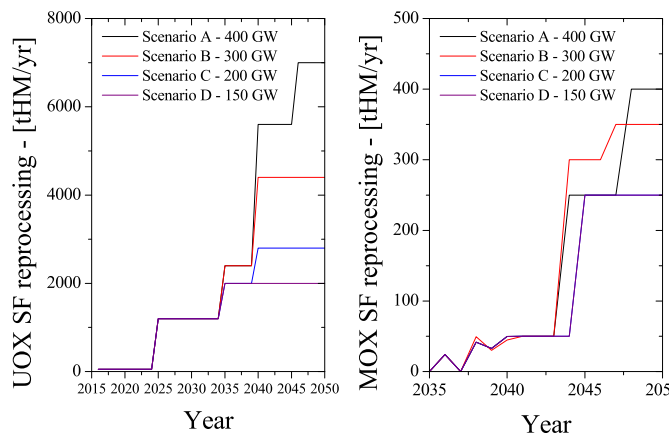


Fig. 11. Reprocessing capacity of UOX (left) and MOX (right) spent fuels.

presented in Fig. 8.

The calculated data fit quite well with the NBS data and IAEA data from 2000 to 2009, but the simulated results are around 20% higher than the NBS data from 2010. Most of the new PWR from 2010 are CPR-1000 which is designed on the basis of M310. According to the IAEA website (IAEA), its power is equal or beyond 1000 MWe. However, the deviation with the NBS data decreases strongly when the nuclear power of M310 (944 MWe) is employed in the calculation to simulate CPR-1000 as shown in Fig. 8. Therefore CPR-1000 was finally set to deliver 944 MWe in the simulations. The cumulated SF calculated by COSI is illustrated in Fig. 9, which does not include SF unloaded from CANDU reactors, since they are not reprocessed.

The spent fuel accumulated from Chinese reactors from 1991 to 2010 is about 1300 tHM according to (Guoqing, 2010). The total mass of SF by the end of the year 2010 is 1408 tHM, which is rather consistent with the data reported in the literature.

4.2. Reprocessing and Pu inventory

In order to keep the Pu inventory as low as possible, notably as regards proliferation issues, the reprocessing capacity increases in concert with the needs in MOX fuel, *i.e.* with the installed power of FBR in the case of the “Pu recycled in FBR” strategy. The proportion of FBR in power ratio is shown in Fig. 10, and the capacities of UOX SF reprocessing and MOX SF reprocessing plants are reported in Fig. 11. The separated Pu stocks for each scenario are shown in Fig. 12.

The results show that FBR may account for around 25–30% of the total installed nuclear capacity by 2050. Such a ratio is an upper limit,

and should however constitute a difficult challenge to reach. Fig. 12 reveals an accumulation of plutonium between 2025 and 2035. Whatever the progress of nuclear power is, the scheduled reprocessing capacity of 1200 tHM/yr in this period is particularly high in all the scenarios: the Pu in excess should be recycled in PWR or the reprocessing capacity reduced during this period. The Pu extracted from PWR SF remains the main resource to support the FBR development, but when no new PWR starts (from 2035 or 2040), the Pu extracted from PWR SF does not suffice to meet the demand of FBR.

4.3. Natural uranium

The one-through way is representative of the current situation in China, because there is no reprocessing plant or commercial FBR yet. This section discusses the results of the calculation as regards the demand of natural uranium, if China would have decided to deploy only PWR following “one-through” way. The results from the different scenarios are illustrated in Fig. 13.

The cumulative demands of natural uranium for each scenario are near 700,000 tons at least and 1,500,000 tons at the most. Assuming uranium resources in China are consumed in priority (265,500 tons, see section 2.1), they dry up in 2030 for scenario D and in 2025 for scenario A. Besides, the consumed natural uranium of each scenario in 2050 will take 11.8%, 14.5%, 19.5 and 24.4% of total low-cost uranium resources in the world respectively (5,902,900 tons, see section 2.1). The price of uranium ore and its derivatives may significantly increase, since the number of PWR should rise all over the world up to 2050.

By the end of the year 2050, the demands of natural uranium for each scenario option are reported in Fig. 14. Compared with the “one-through” option, the cumulative consumption of natural uranium is reduced in “Pu recycled in PWR” and “Pu recycled in FBR” options.

Uranium savings are shown in Table 3. This table reveals rather low uranium savings by 2050. When the fleet will be composed of fast reactors only, as in (Gao et al., 2017), China could however emancipate totally from natural uranium resources. But the last PWR shall not be shut down before 2095 or 2100 if they are in operation for 60 years. The transition period toward the independence from uranium ore appears actually very long, which explains relatively low uranium savings by 2050. These results plead thus for a fast FBR deployment in China, since the natural uranium consumption will continue long after 2050 even in this case, implying a considerable tension on this resource by the end of the 21st century. The transition toward a closed fuel cycle should start as soon as possible in these conditions. If tensions nonetheless appear on uranium resources, recycling plutonium once in PWR as in France could lead to significant savings at competitive costs (Gao et al., 2017).

4.4. Spent fuel

If China would follow the “one-through” way to develop nuclear energy, the cumulative SF stored in on-site pool and eventually in geological disposals would increase dramatically, as illustrated in Fig. 15.

The cumulative SF stored in on-site pool by the end of 2050 will be 13,000 tons at least and 32,000 tons at the most, which means that numerous pools would have to be built and maintained for storage purpose. The cumulative SF by the end of 2050 which have to be permanently stored (offsite) weight 55,000 tons at least and 99,000 tons at the most. As long as the “one-through” cycle will apply in China, SF storage will remain a prime concern. For a same electrical production curve, the SF accumulation is different among the three recycling options. Taking Scenario D and Scenario A as examples, the SF stocks are shown in Fig. 16.

The results reveal that the reprocessing plant with the capacity of 1200 tHM/yr from 2025 cannot stop the increase of interim stored SF. It is necessary to increase the capacity of reprocessing in order to

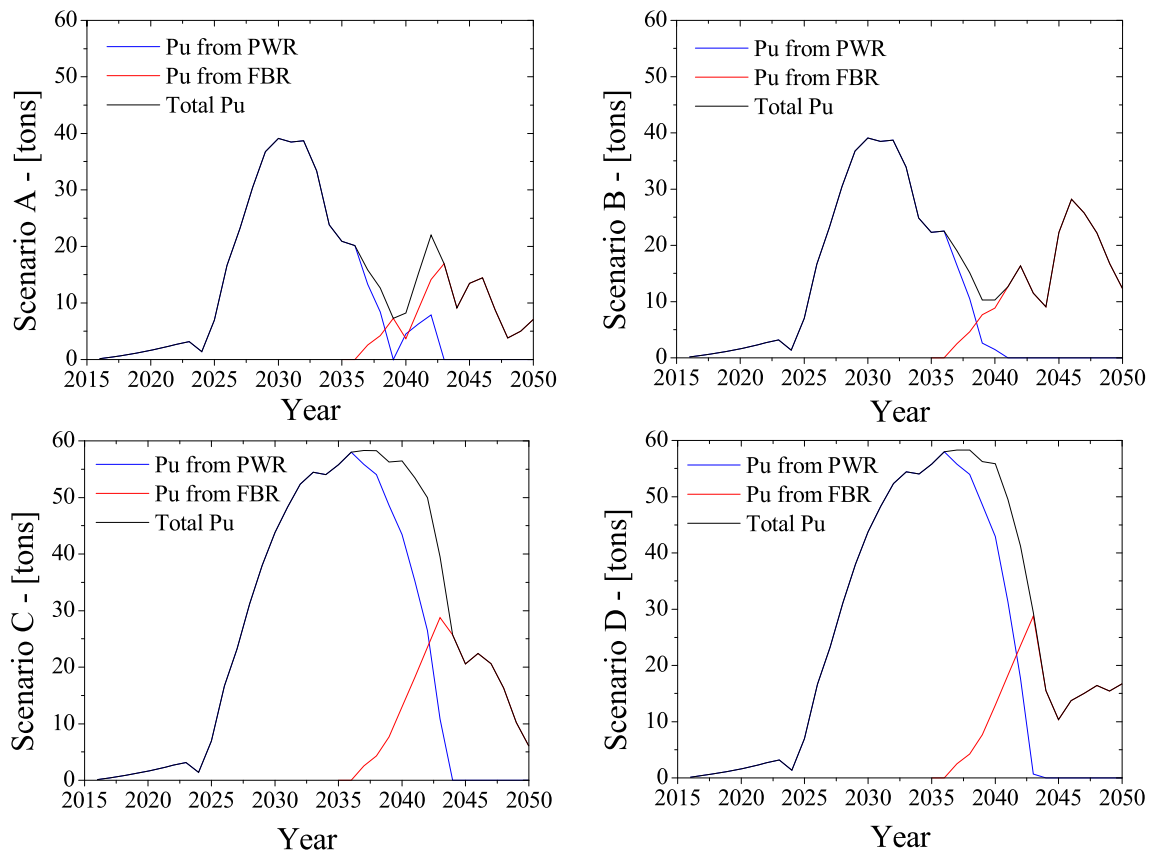


Fig. 12. Separated Pu stocks for “Pu recycled in FBR” scenarios.

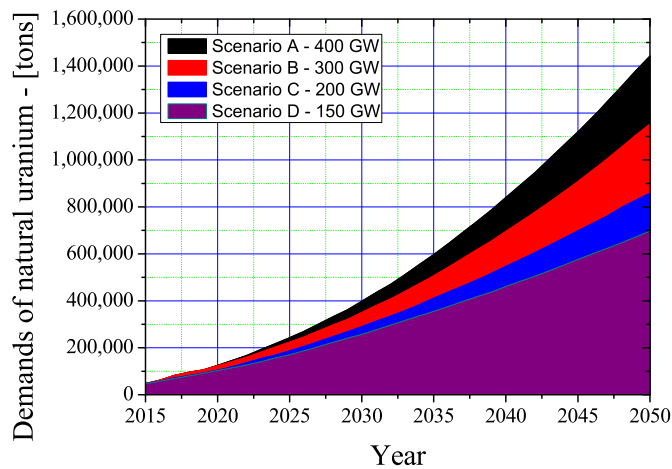


Fig. 13. Demands of natural uranium for “one-through” way.

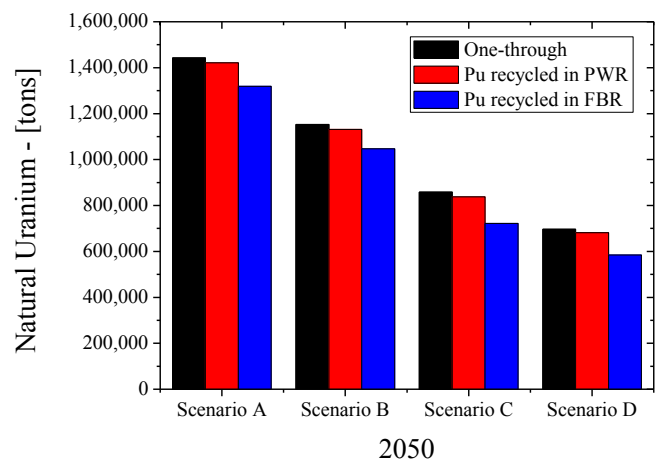


Fig. 14. Comparison of natural uranium consumptions with respect to the various scenarios.

Table 3
Relative natural uranium savings with respect to one-through cycle by 2050.

Scenario	Pu recycled in PWR (%)	Pu recycled in FBR (%)
A	1.5	7.1
B	1.9	7.4
C	2.2	13.5
D	2.4	14.0

significantly reduce the mass of interim stored SF. Besides, the reprocessing capacity should correspond to nominal operations of reprocessing plants of reasonable lifetime. In other words, it is not realistic to consider oversized reprocessing plants with high capacity drying up all SF in several years. The spent fuel in interim storage for all the scenarios with respect to the 3 different options are presented in Table 4.

4.5. Waste disposal

If the “one-through” way would apply in the future in China, there would be no reprocessing, so that the cumulative SF stored in off-site pool shown in Fig. 15 might be sent into geological disposal, since they contain long-life radioactive elements: SF would constitute the ultimate

solid wastes. To the opposite, reprocessing implies the separation of minor actinides (MA) and fission products. These elements are eventually confined into high level waste (HLW) glass packages which

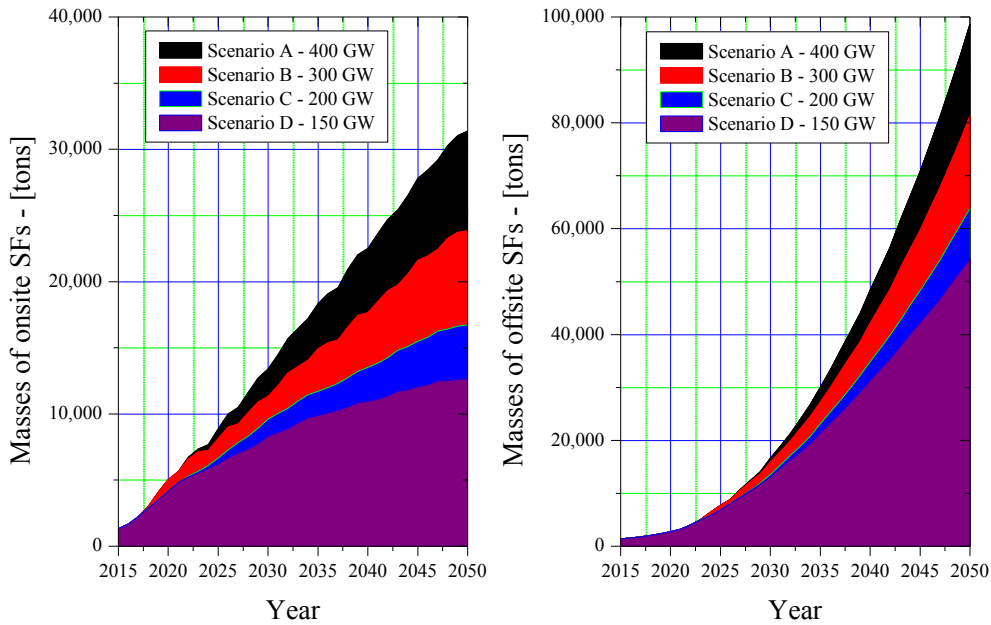


Fig. 15. SF stocks for "one-through" cycle (OTC) scenarios.

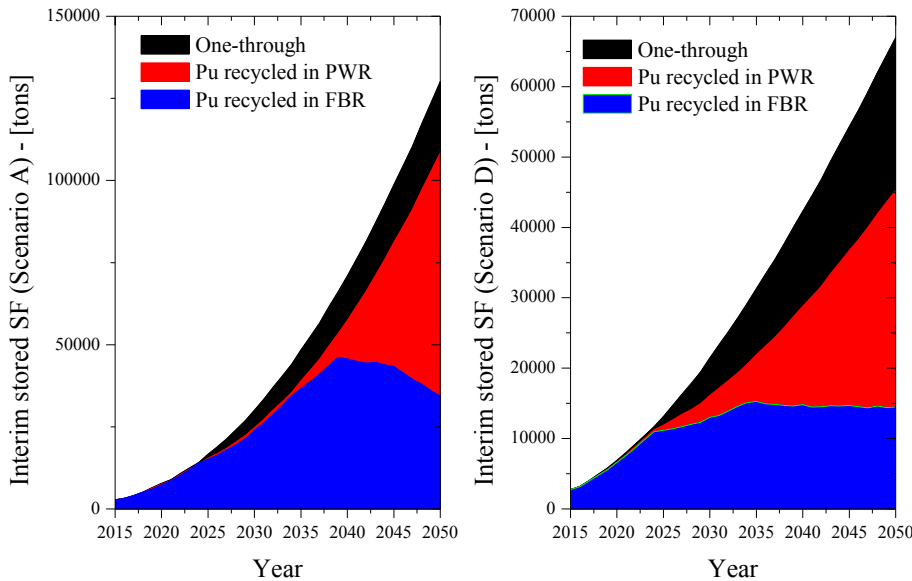


Fig. 16. Comparison of SF stocks with different options.

Table 4
SF in interim storage in 2050.

Scenario	OTC	Pu recycled in PWR		Pu recycled in FBR	
	Absolute Value (tons)	Absolute Value (tons)	Decrease from OTC (%)	Absolute Value (tons)	Decrease from OTC (%)
A	130,000	108,000	17%	46,000	65%
B	105,000	84,000	20%	33,000	69%
C	81,000	59,000	27%	21,000	74%
D	67,000	45,000	33%	15,000	78%

therefore constitute the ultimate solid wastes to manage for long. In this case, uranium and plutonium are recycled into new MOX fuels. Thanks to this, the amount of solid wastes can be considerably reduced when FBR are massively deployed, since a high reprocessing capacity is concomitantly setup, as shown in Fig. 17 for Scenarios D and A. In this figure, for the "one-through" cycle scenario, the mass of solid wastes

which shall be eventually disposed for a long time is the mass of heavy atoms (expressed in tHM). This is not the case of glass packages, for which the total mass is reported.

Finally, more FBR are deployed, more SF are recycled into MOX fuel. FBR therefore contribute to empty SF storage facilities in one hand, but significant reprocessing capacities must be deployed to supply them in the other hand.

If MA (Np, Am and Cm) were separated from HLLW and transmuted within ADS or FBR (assuming a 100% efficiency of the separation and transmutation processes), the mass and the thermal power of HLW packages would decrease: the results in 2050 are shown in Table 5.

It can be inferred from Table 3 that MA separation would decrease the mass of ultimate solid wastes by around 3.2%, and the amount of glass units by around 3.6%. The performance of MA separation and transmutation appears finally rather poor when these technologies apply at the scale of a nuclear fleet. Toxicity by ingestion would also decrease slightly, as shown in Fig. 18.

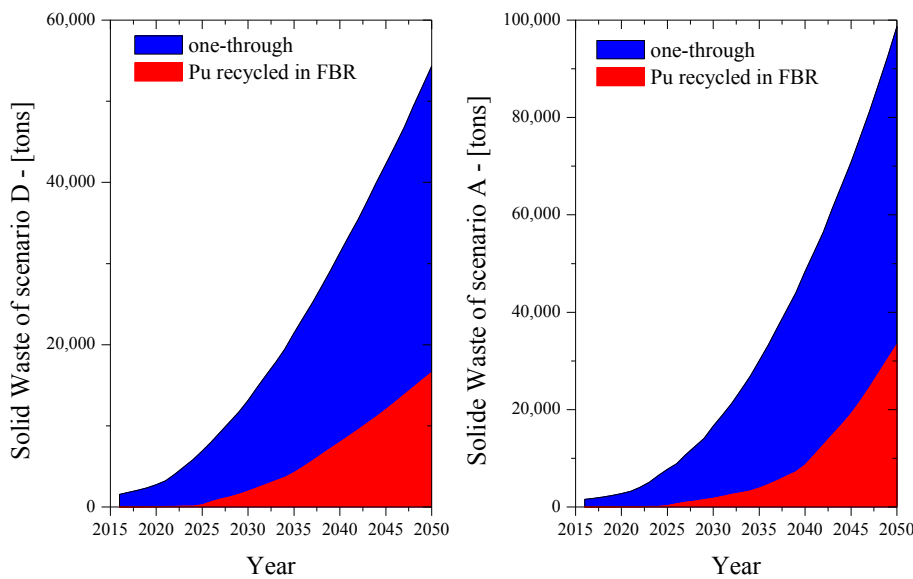


Fig. 17. Comparison of solid waste production.

Table 5

Comparison between the glass units produced with MA separation (MA SP) and no separation (No SP) in scenarios with Pu recycled in FBR by the end of 2050.

Scenario	Thermal power (kW)		Number of Glass units		Total Masses (tHM)	
	MA SP	No SP	MA SP	No SP	MA SP	No SP
A	13,260	15,770	65,570	67,850	4590	4740
B	8320	9950	50,730	52,601	3630	3750
C	5370	6430	38,510	39,960	2700	2790
D	3960	4800	32,450	33,710	2270	2350

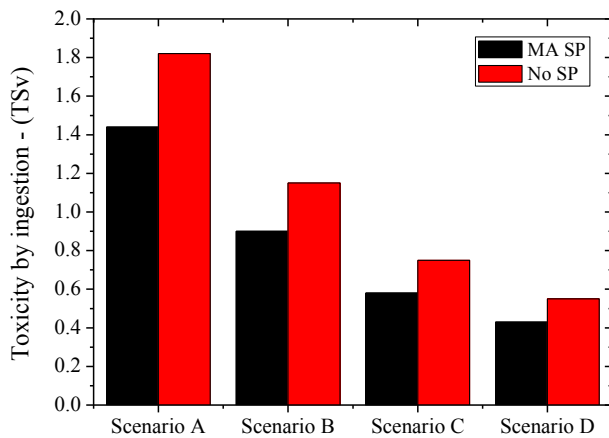


Fig. 18. Toxicity by ingestion of wastes for the various scenarios in 2050.

5. Conclusion

The development of nuclear power in China is a part of the solution to the atmospheric pollution and to the growing electricity demand in this country, since in China, the public attitude as regards nuclear power is positive (Wu, 2017). However, nuclear power currently represents only few percents of the electricity production. It therefore needs a fast development in the next decades, supported by the Chinese government.

If China would go on relying on a “one-through” cycle, the foreseen Chinese nuclear expansion would need a huge amount of natural uranium and generate a substantial amount of spent fuel by 2050. Therefore, the closed fuel cycle appears as a necessary political and strategic decision. The results herein show that the selection of the

future strategy should take into account aspects that go beyond the natural uranium requirement. The capacity of fuel fabrication and reprocessing plants, the timetable and the amplitude of the FBR development, as well as proliferation concerns related to sensitive elements, need to be part of the discussion.

It seems impossible for China to start two BN-800 fast reactors before 2020 since the current pilot reprocessing plant of 50 tHM/yr cannot produce enough Pu. The deployment of FBR can start only from 2035 for scenario C and D (faster nuclear power growths), and from 2040 for scenario A and B. The pace of FBR development should be controlled carefully by the proportion of FBR and PWR with respect to the reprocessing capacity. The performances of MA separation and transmutation appear finally rather poor when these technologies apply at the scale of a nuclear fleet.

Compared with the “one-through” way, the “Pu recycled in FBR” strategy can significantly decrease the consumption of natural uranium, the mass of SF and then the interim storage capacity needed (pools). It also leads to a reduction of the mass of solid wastes. A fast FBR deployment constitutes however the best long-term strategy for China in terms of sustainable development of nuclear energy.

However, natural uranium savings appear rather low by 2050, because the transition toward a fast reactor fleet independent from uranium ore lasts decades. In these conditions, starting to close the fuel cycle is a priority, since this might be the only way to avoid the shortage of uranium resources in the end. A closed fuel cycle may be reached by the end of the century, if the first corners in this way are turned for the next decade. In this case, the fraction of FBR in 2050 may stand at the most for around 30% of the total nuclear power in China.

Once investments in reprocessing technologies will have been decided, separated Pu can also be recycled in PWR in the form of MOX fuel, since this leads to short-term natural uranium savings and reduction of spent fuel stocks. Recycling plutonium into PWR MOX fuel

may therefore constitute a reliable strategy if FBR are delayed.

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